

# History of research into confinement improvement

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## Special issue:

Plasma physics in the 20th century as told by players

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 PHYSICAL JOURNAL H

## The history of research into improved confinement regimes

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**Abstract.** Increasing the pressure by additional heating of magnetically confined plasmas had the consequence that turbulent processes became more violent and plasma confinement degraded. Since this experience from the early 1980ies, fusion research was dominated by the search for confinement regimes with improved properties. It was a gratifying experience that toroidally confined plasmas are able to self-organise in such a way that turbulence diminishes, resulting in a confinement with good prospects to reach the objectives of fusion R&D. The understanding of improved confinement regimes revolutionized the understanding of turbulent transport in high-temperature plasmas. In this paper the story of research into improved confinement regimes will be narrated starting with 1980.

<b>Highlight</b> <b>Open Access</b>
<p>Editorial  <b>Editorial introduction to the special issue “Plasma physics in the 20th century as told by players”</b>            Patrick H. Diamond, Uriel Frisch and Yves Pomeau            Published online: 30 November 2018            DOI: 10.1140/epjh/e2018-90061-5            Abstract   PDF (327.0 KB)</p>
<p>Oral history interview  <b>An interview with Roald Sagdeev: his story of plasma physics in Russia, 1956–1988</b>            Roald Z. Sagdeev and Patrick H. Diamond            Published online: 23 October 2018            DOI: 10.1140/epjh/e2018-90042-3            Abstract   PDF (434.6 KB)</p>
<p><b>From thermonuclear fusion to Hamiltonian chaos</b>            D.F. Escande            Published online: 19 January 2017            DOI: 10.1140/epjh/e2016-70063-5            Abstract   PDF (585.7 KB)</p>
<p><b>Wave-particle and wave-wave interactions in hot plasmas: a French historical point of view</b>            Guy Laval, Denis Pesme and Jean-Claude Adam            Published online: 01 November 2016            DOI: 10.1140/epjh/e2016-70050-2            Abstract   PDF (1.101 MB)</p>
<p><b>The Joint European Torus (JET)</b>            Paul-Henri Rebut            Published online: 27 February 2017            DOI: 10.1140/epjh/e2017-70068-y            Abstract   PDF (3.002 MB)</p>
<p><b>Strong turbulence, self-organization and plasma confinement</b>            Akira Hasegawa and Kunioki Mima            Published online: 26 October 2018            DOI: 10.1140/epjh/e2018-90033-4            Abstract   PDF (1.285 MB)</p>
<p><b>Open Access</b>  <b>The history of research into improved confinement regimes</b>            F. Wagner            Published online: 05 January 2017            DOI: 10.1140/epjh/e2016-70064-9            Abstract   PDF (1.154 MB)</p>
<p><b>The large tokamak JT-60: a history of the fight to achieve the Japanese fusion research mission</b>            Mitsuru Kikuchi            Published online: 23 November 2018            DOI: 10.1140/epjh/e2018-90054-2            Abstract   PDF (3.898 MB)</p>

# Importance of the energy confinement time $\tau_E$

A burning fusion reactor has to meet the  $\langle n \rangle \langle T \rangle \tau_E$  – condition:  $\sim 10^{20} \text{ m}^3 \text{ 10 keV 3 sec}$

→ a high confinement time  $\tau_E$  is the key to a fusion reactor

Power balance under steady-state:  $\frac{3}{2} \langle n \rangle \langle T \rangle \text{Vol} = (P_{\text{fus}} + P_{\text{aux}}) \tau_E$

Confinement and transport:  $W/\tau_E = -n \chi \text{ grad}T \text{ surface} \rightarrow \tau_E \sim a^2/\chi$

$\chi$  is the heat diffusivity

heat is transported via Coulomb-collisions and by turbulence

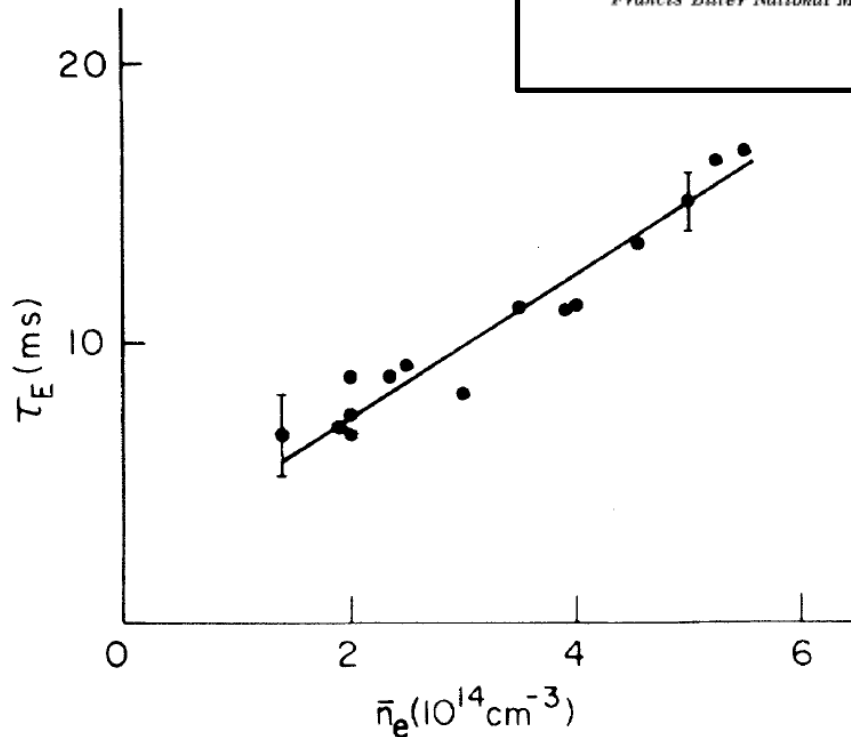
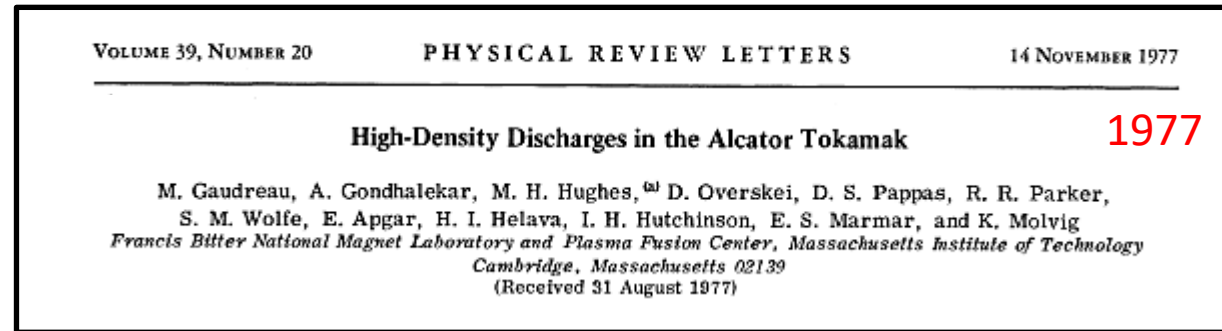
apart from exotic conditions,  $\tau_E$  is governed by plasma turbulence (also  $\tau_p, \tau_\Phi$ )

Though there is tremendous progress in understanding transport, predictions are still vague – e.g. what will be the confinement time of ITER?

→ statistical analysis and empirical scaling rules provide  $\tau_E$  values

# The first $\tau_E$ scaling of relevance: Alcator-A ohmic scaling

Neo-Alcator scaling: Middle of the 70ies

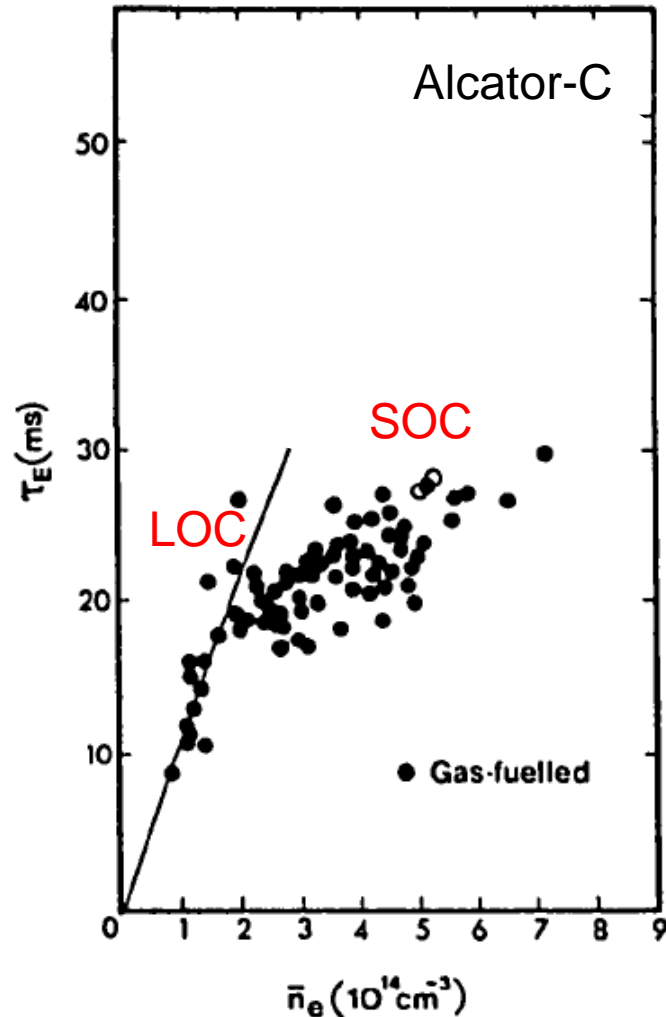


$$\tau_E \sim n_e$$

favourable scaling because  
fusion product  $n \tau_E \sim n^2$

# The first disappointment: the SOC regime

$\tau_E$  saturates toward higher densities



First communicated **1978**:

GONDHALEKAR, A., GRANETZ, R., GWINN, D., HUTCHINSON, I., KUSSE, B., et al., in Plasma Physics and Controlled Nuclear Fusion Research 1978 (Proc. 7th Int. Conf. Innsbruck, 1978), Vol.1, IAEA, Vienna (1979) 199.

# The causes for the $\tau_E$ -saturation

This question needs the understanding of turbulent transport

energy is transported by potential fluctuations in the plasma causing fluctuating drifts

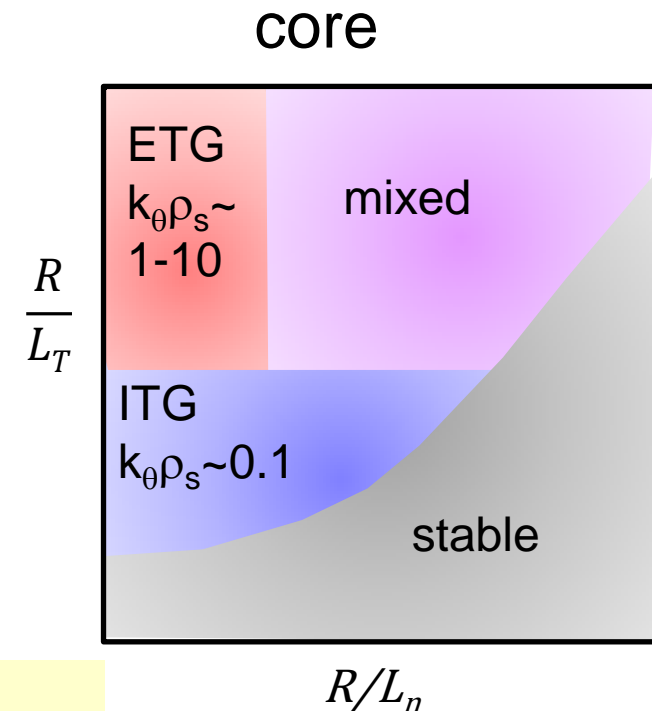
basic electrostatic instabilities : **drift waves**

## Driving forces: gradients

$\nabla T_e$ : ETG (electron-temp. gradient-mode, core)

$\nabla T_i$ : ITG-mode (ion-temp. gradient-mode, core/edge)

$\nabla n$ : TEM (trapped electron mode, edge)



$k_\theta$  = poloidal eddy wave vector  
 $\rho_s$  = Larmor radius at ion sound velocity  
 $R$  = major radius  
 $L_n = n/\nabla n$ : density gradient length

after J. Weiland

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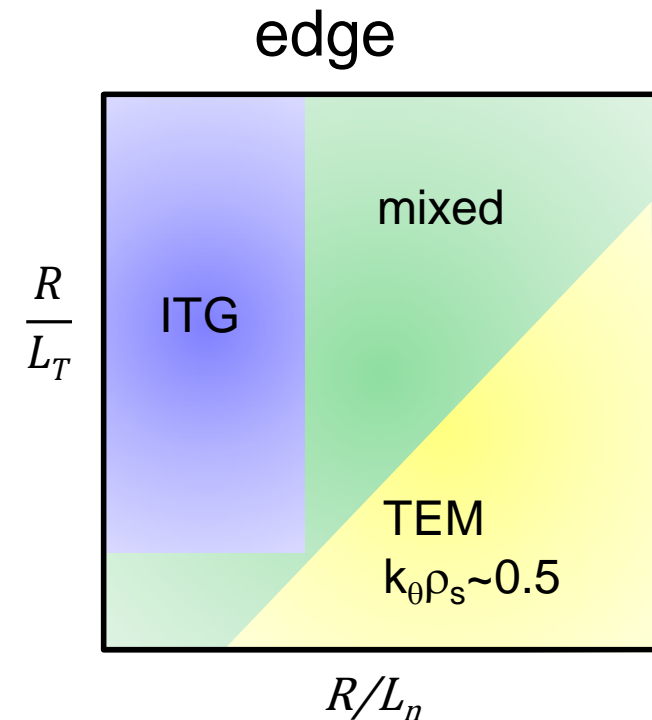
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## Reality more complex:

critical onset gradients

marginal stability conditions

profile resilience

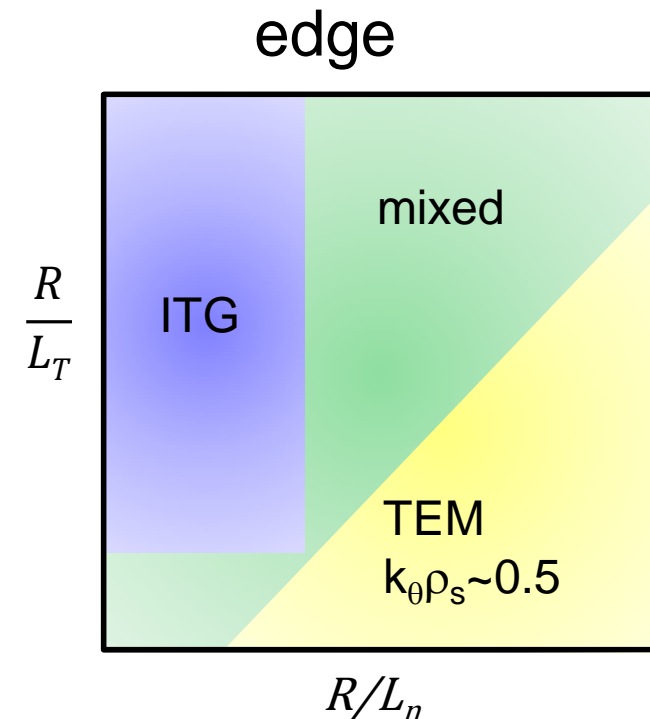
development of large radial scales (streamers)

zonal flows (ZF, GAMs)

## Important parameters:

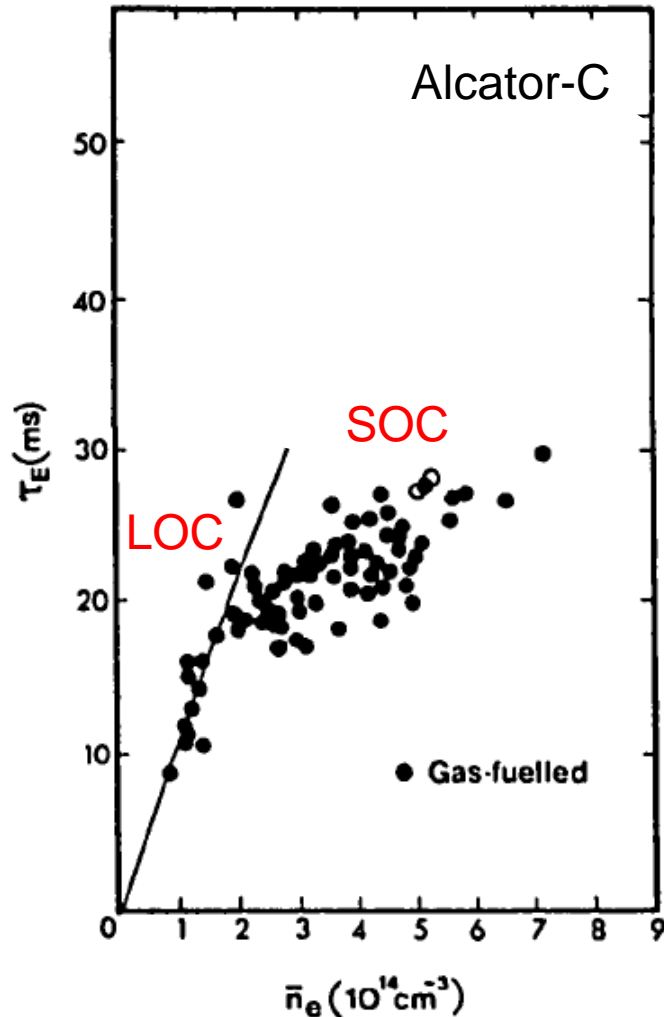
$q$ , shear, reversed shear

collisionality,  $Z_{\text{eff}}$ ,  $T_e/T_i$ , fast particles



after J. Weiland

# The causes for the $\tau_E$ -saturation



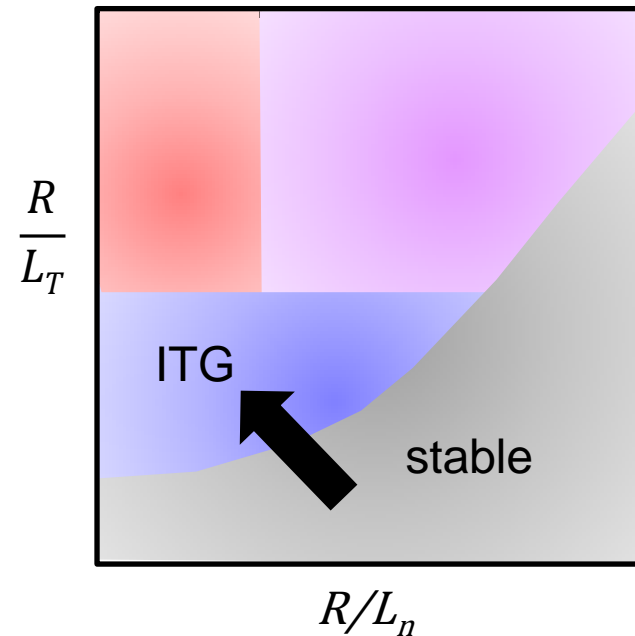
## Towards higher density:

Ions are heated by electron collisions

Ion gradient increases

Density profile becomes flatter (source to edge)

ITG turbulence is enhanced



30 years later:

core toroidal rotation changes direction at  
LOC  $\rightarrow$  SOC

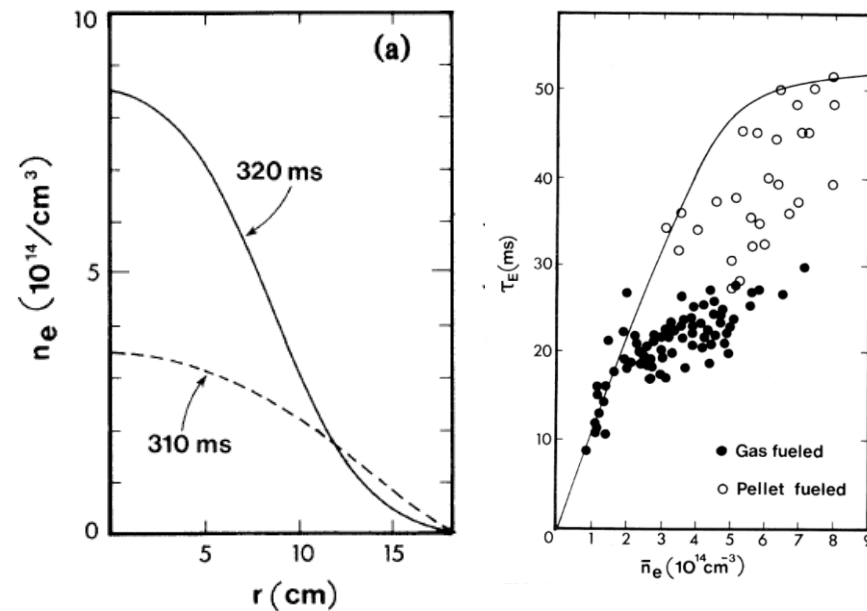
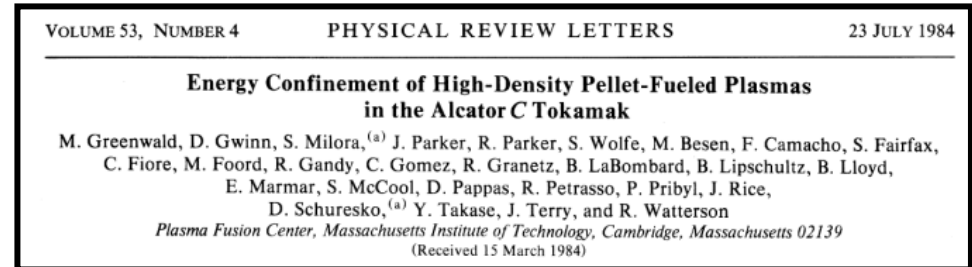
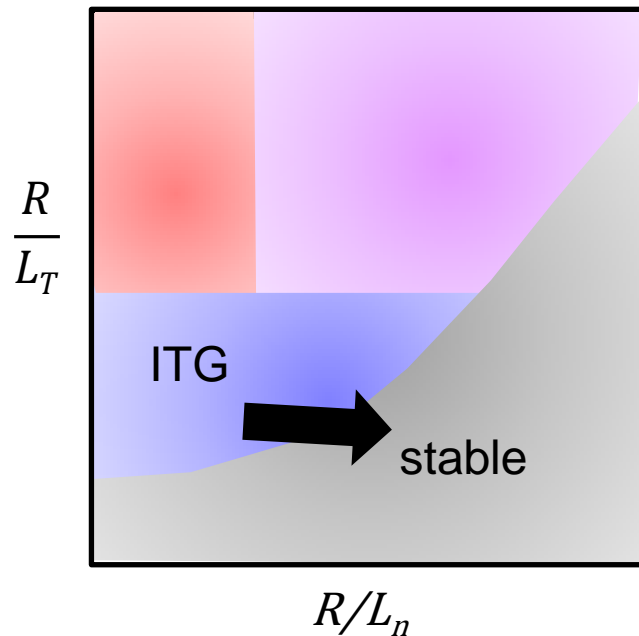


# Ways to overcome this problem: external

## Pellet fuelling

## 1984

Particle source to core  
by pellet fuelling



important:  
the improved state can be initiated

# Ways to overcome this problem: internal

## IOC-mode of ASDEX, 1988

VOLUME 61, NUMBER 9

PHYSICAL REVIEW LETTERS

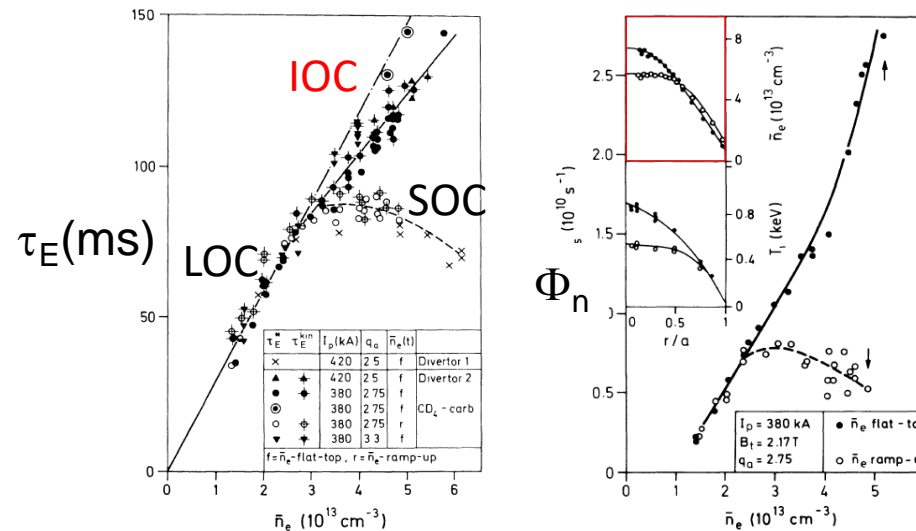
29 AUGUST 1988

### Improved Confinement in High-Density Ohmic Discharges in ASDEX

F. X. Söldner, E. R. Müller, F. Wagner, H. S. Bosch, A. Eberhagen, H. U. Fahrbach, G. Fussmann, O. Gehre, K. Gentile,<sup>(a)</sup> J. Gernhardt, O. Gruber, W. Herrmann, G. Janeschitz, M. Kornherr, K. Krieger, H. M. Mayer, K. McCormick, H. D. Murmann, J. Neuhauser, R. Nolte, W. Poschenrieder, H. Röhr, K.-H. Steuer, U. Stroth, N. Tsois,<sup>(b)</sup> and H. Verbeek

Max-Planck-Institute für Plasmaphysik, EURATOM Association, D-8046 Garching bei München, West Germany

(Received 16 May 1988)



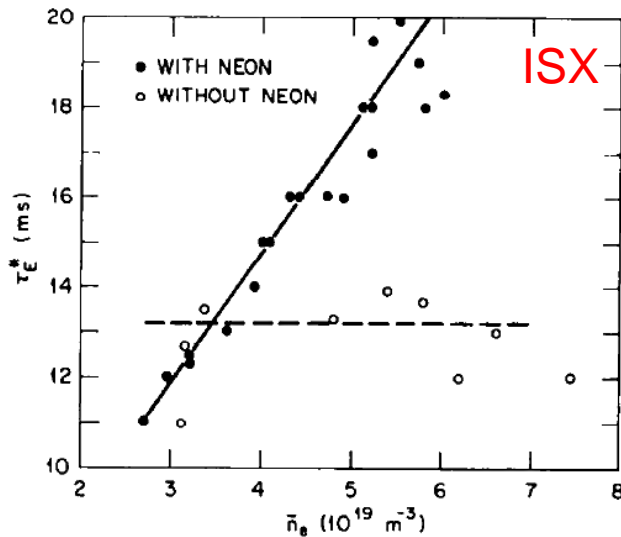
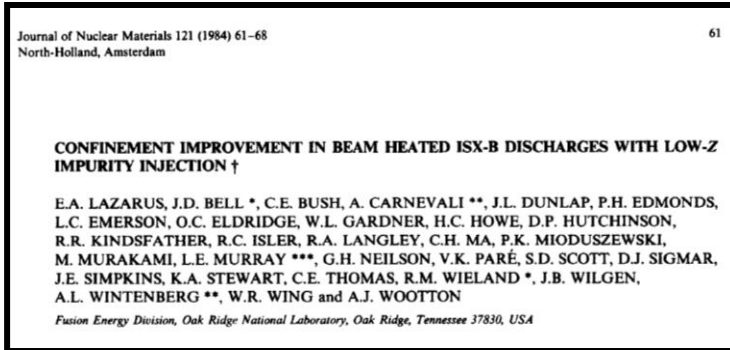
Self-induced improvement

Different branches: SOC-IOC

trick: change edge conditions by reducing edge fuelling

# Ways to overcome this problem

Z-mode (ISX)-B-mode (T10)-RI-mode (TEXTOR)



→ effect of  $L_n$  on ITG stability robust

cause:

$Z_{\text{eff}}$  reduces the growth of the ITG instability

# A new epoch: auxiliary heating

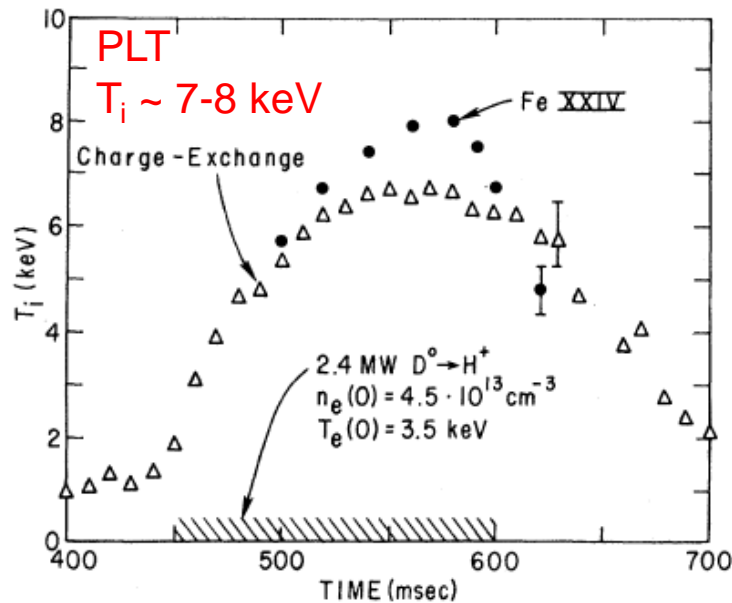
In the 70ies, start of auxiliary heating in the MW-range. Why?  $\sigma_{ei} \sim T_e^{3/2}$

VOLUME 43, NUMBER 4      PHYSICAL REVIEW LETTERS      23 JULY 1979

**Neutral-Beam-Heating Results from the Princeton Large Torus**

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 N. Sauthoff, G. Schilling, J. Schivell, G. Schmidt, F. Stauffer,<sup>(c)</sup> L. Stewart,<sup>(c)</sup>  
 W. Stodiek, R. Stooksberry,<sup>(d)</sup> J. Strachan, S. Suckewer, H. Takahashi,  
 G. Tait,<sup>(e)</sup> M. Ulrickson, S. von Goeler, and M. Yamada  
*Plasma Physics Laboratory, Princeton University, Princeton, New Jersey 08544*  
 and  
 C. Tsai, W. Stirling, W. Dagenhart, W. Gardner, M. Menon, and H. Haselton  
*Oak Ridge National Laboratory, Oak Ridge, Tennessee 37830*  
 (Received 1 March 1979)

1979



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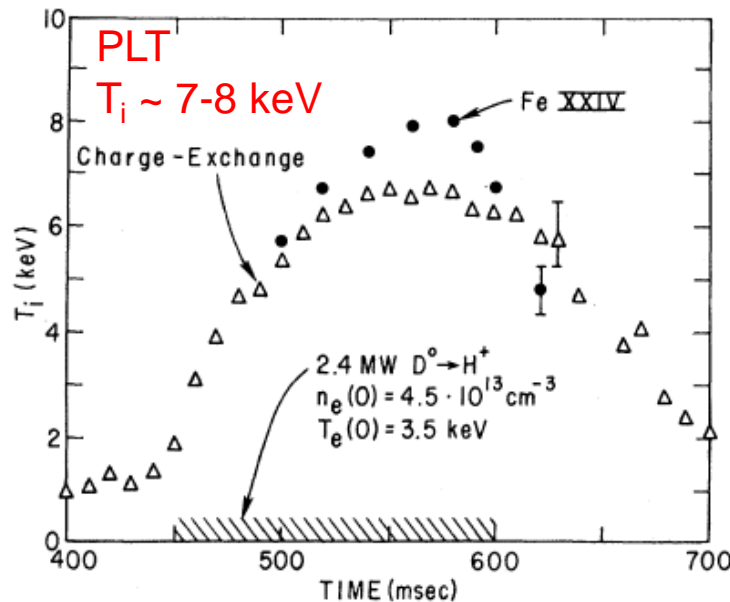
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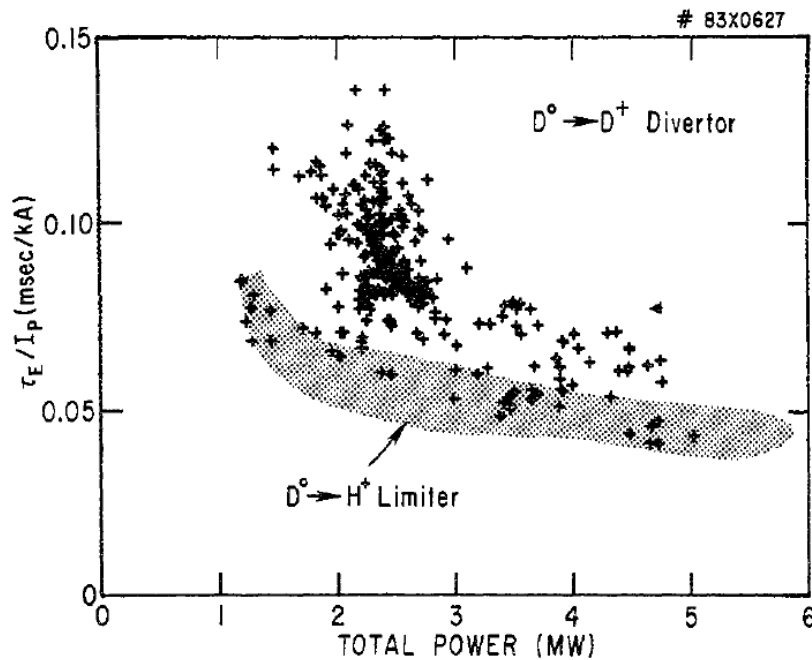
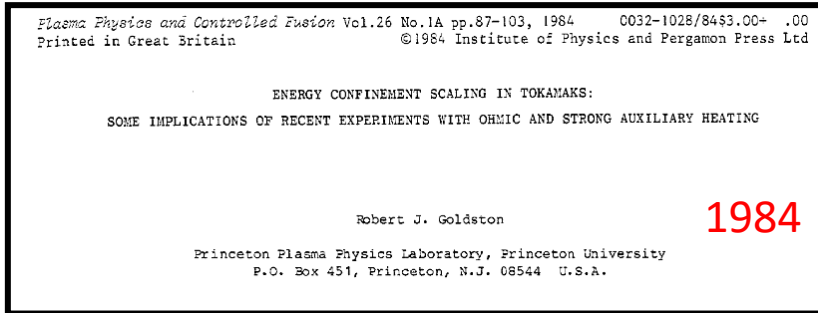
When these results were reported at the 1978 IAEA FE-conference, B. Kadomtsev praised them:

*"I congratulate you (R. Goldston) and the Princeton team on the very impressive achievement of reaching high ion temperatures and penetrating far into the collisionless region - a very important achievement for future reactor applications."*



# The anti-climax: power degradation in the L-mode

beginning of the 80ies

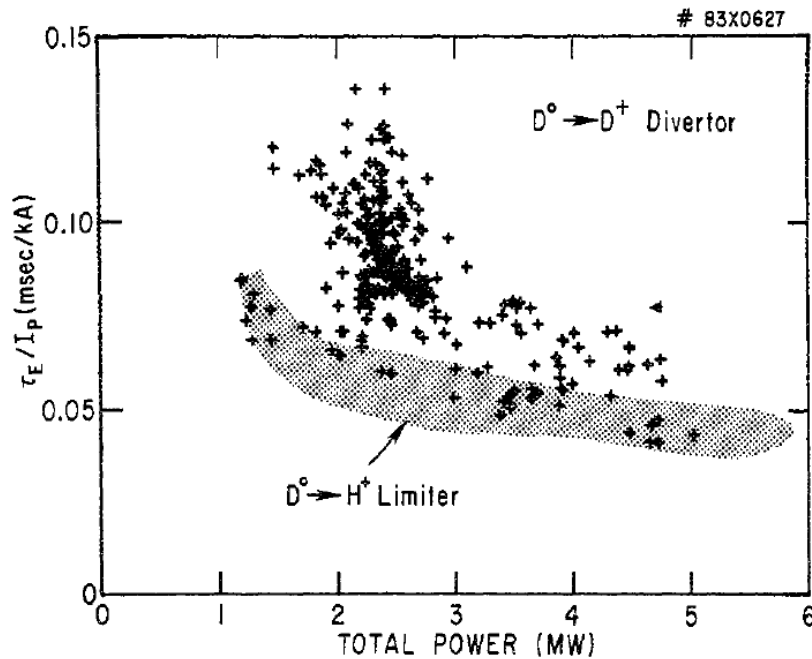
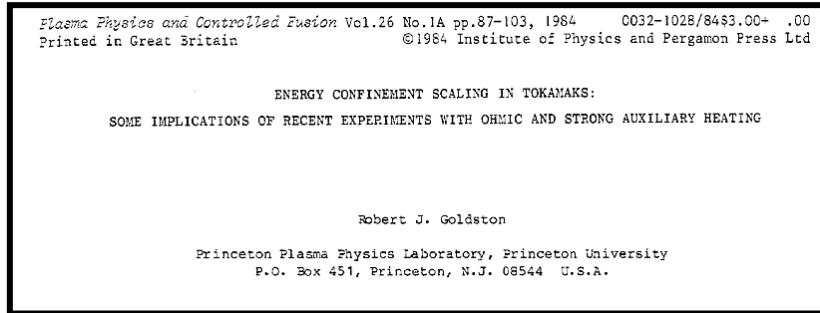


Rob Goldston



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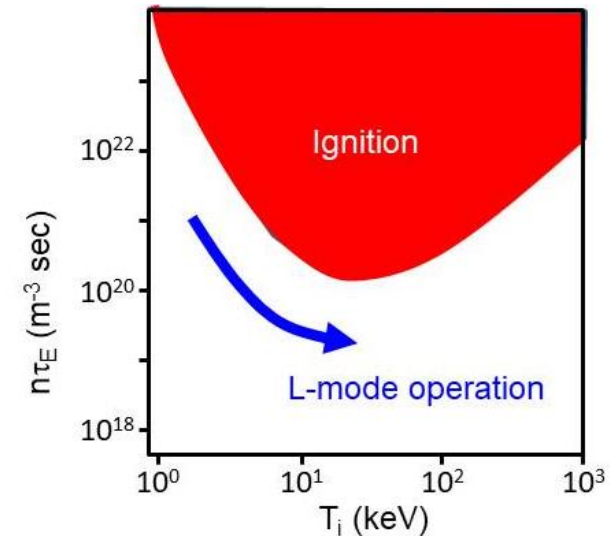
beginning of the 80ies



In the discussion phase of the 11<sup>th</sup> IAEA conference, P-H Rebut, then director of JET, was cited talking about a “lack of significance of auxiliary heating”

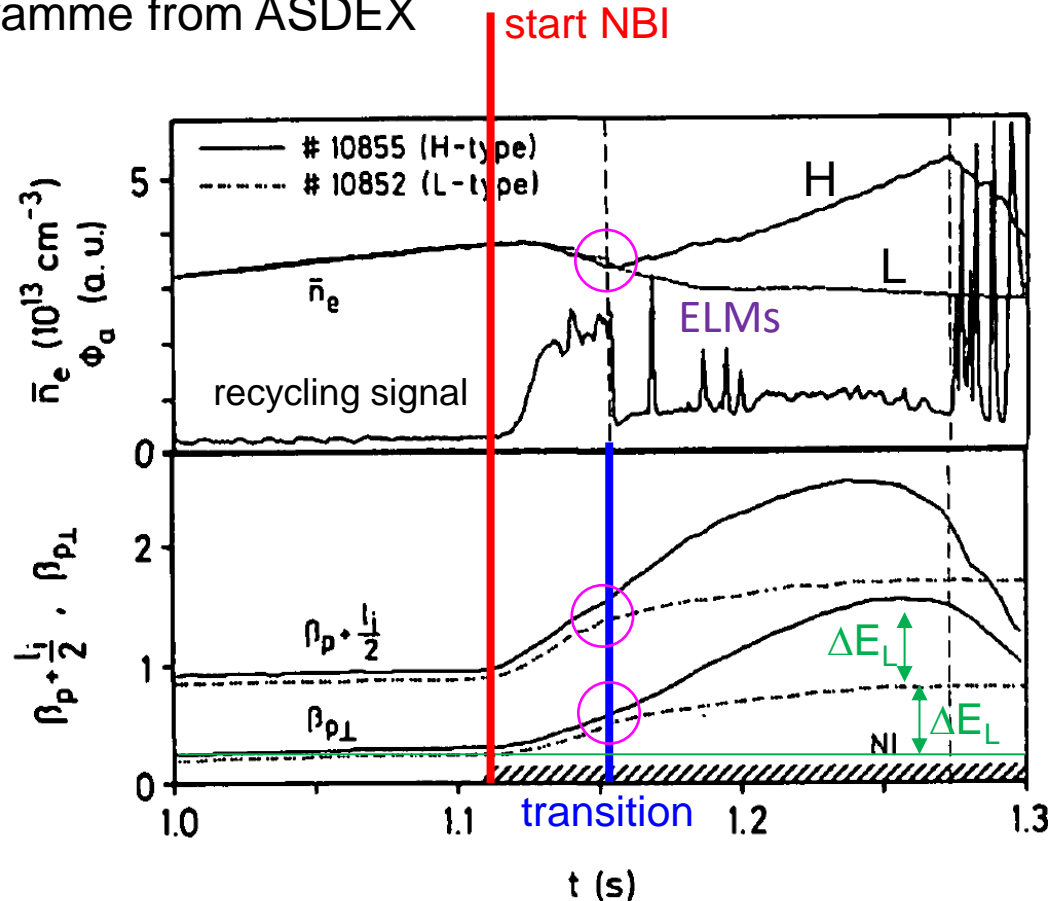


What did he mean?



# The rescue (?): The H-mode of ASDEX in 1982

Historical diagramme from ASDEX



The main features of the H-mode:

- a spontaneous and distinct transition during the heating phase

- both energy- and particle confinement time increase

- a power threshold  $P_{\text{thr}}$  has to be overcome

- new instabilities appear in the H-phase: ELMs, edge-localised modes



# Initial observations and relevance

**(1) L- and H-modes differ in energy confinement time by about a factor of two:**

energy, particle, impurity, and momentum confinement improve simultaneously;  
two operational branches exist; the space in between is not accessible.

**(2) The H-mode transition has a power threshold  $P_{thr}$ :**

Obviously, a critical condition has to be met.

**(3) There is a dwell time after the heating power has been increased from the ohmic level, before the plasma transits into the H-phase:**

A formation process has been initiated by stepping up the heating power with a time scale depending on external settings.

**(4) When the heating power has been switched off, the plasma remains in the H-phase again for a dwell time:**

Also the back transition is not gradual but occurs in a distinct step – the gap between H- and L-mode branches.

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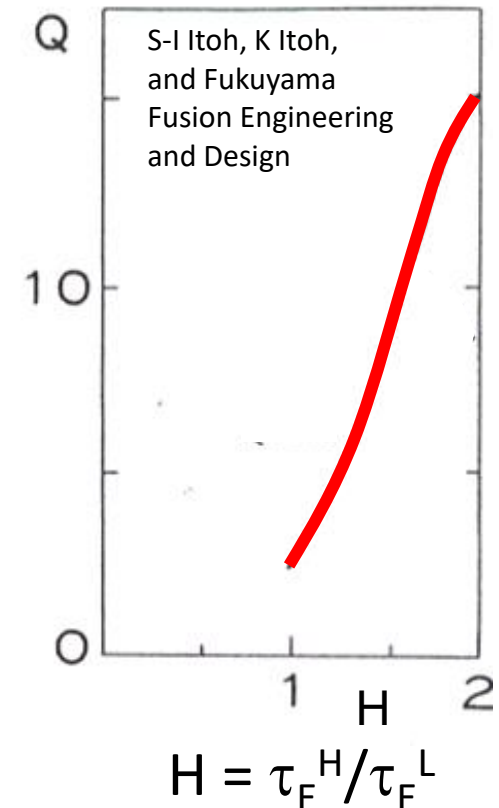
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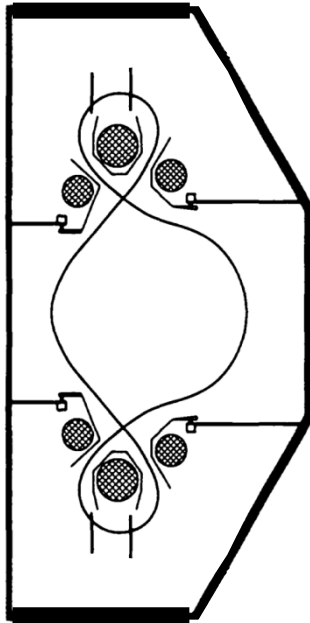
Also the back transition is not gradual but occurs in a distinct step – the gap between H- and L-mode branches.

the benefit  
of improved  
confinement



# Why was the H-mode discovered in ASDEX?

1. Divertor plasma with magnetic separatrix



2. the heating power was increased from 1.8 to 3 MW

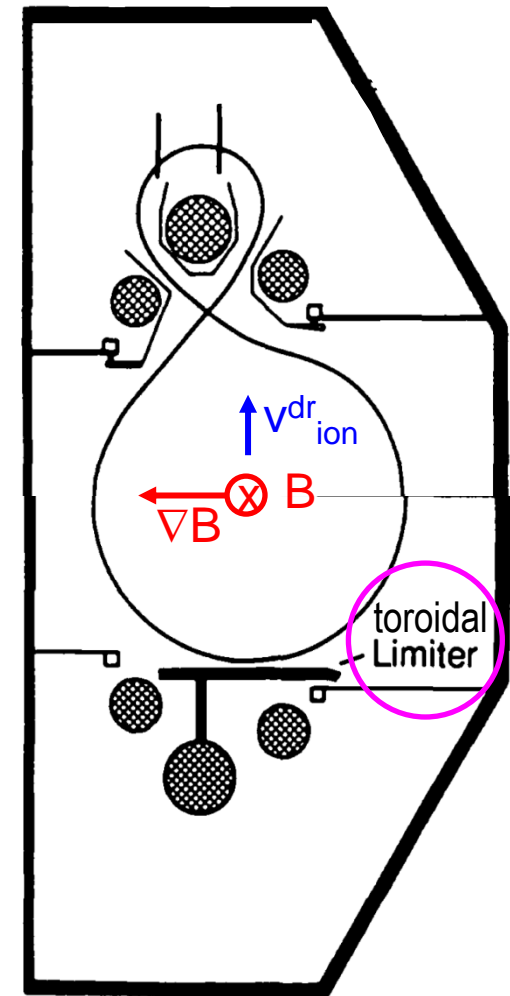
3. Operation has been restricted to upper single-null operation

A **toroidal limiter** was installed

Only upper single-null (SN) operation was possible.

Fortuitously, the  $B \times \nabla B$  ion drift was to the active X-point.

**Thus, the power threshold was reduced**



# The spreading of the H-mode

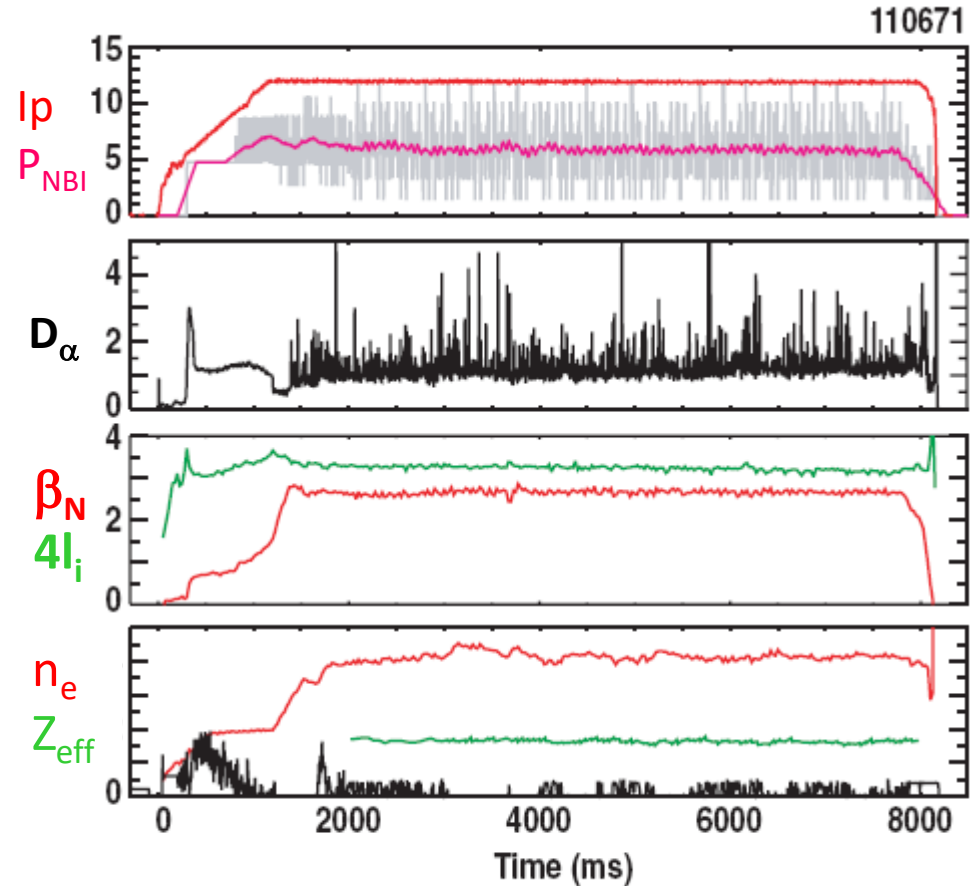
Shortly after ASDEX:

PDX (1984)

then DIIID (then Doublet, 1984)

Much later:

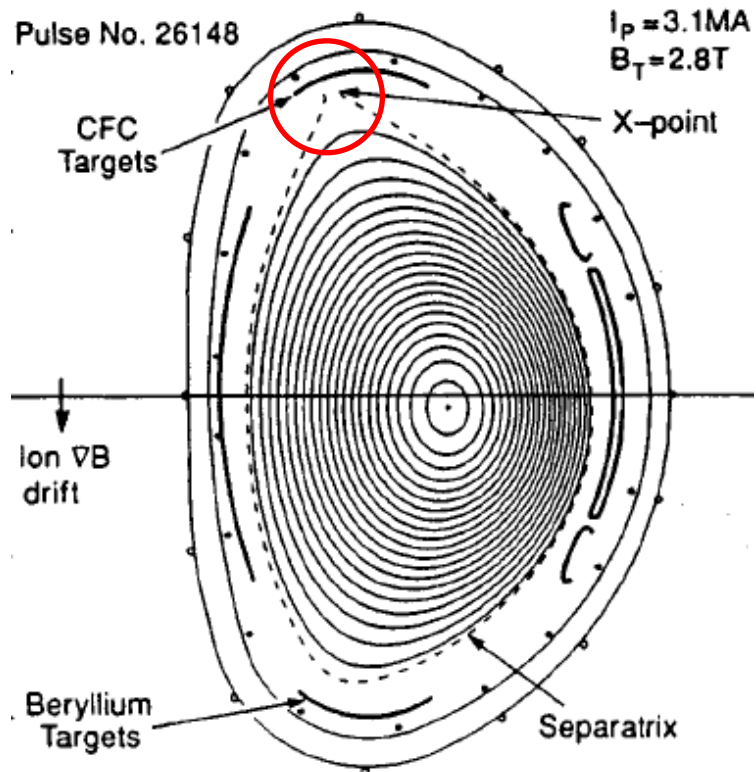
~ 6 s high-performance H-mode discharge of DIII-D



ELMy-H-mode becomes the  
the basis for ITER

# The special story of JET

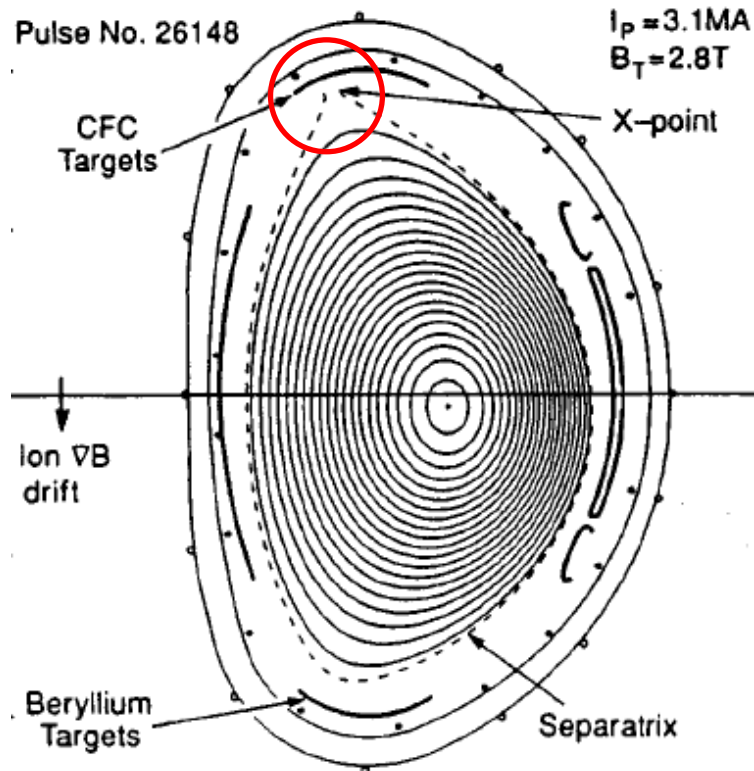
a limiter device with  
marginal X-point (1985)



“illegal” development  
by late Arturo Tanga

# The special story of JET

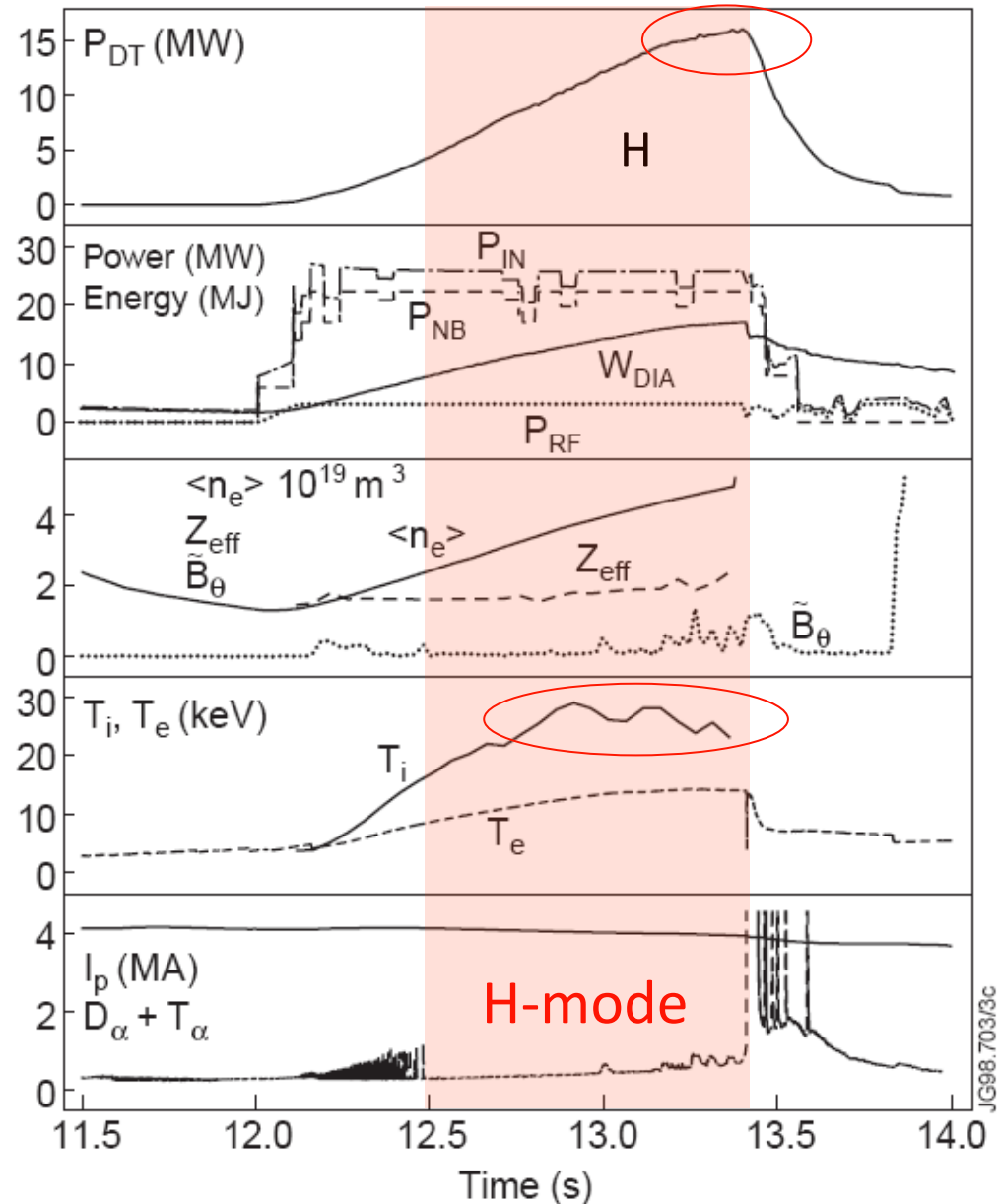
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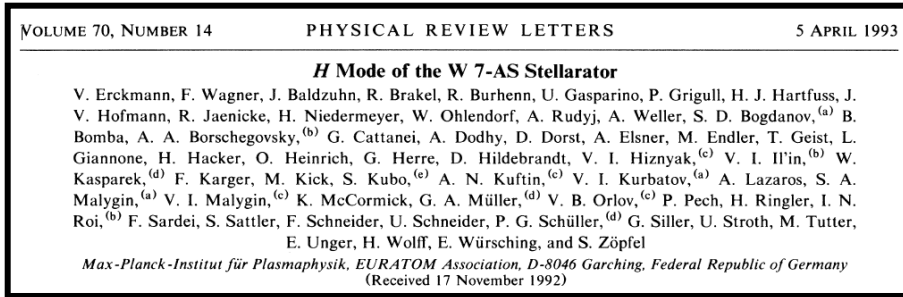
“illegal” development  
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Much later:

The 16.1 MW DT H-mode discharge of JET

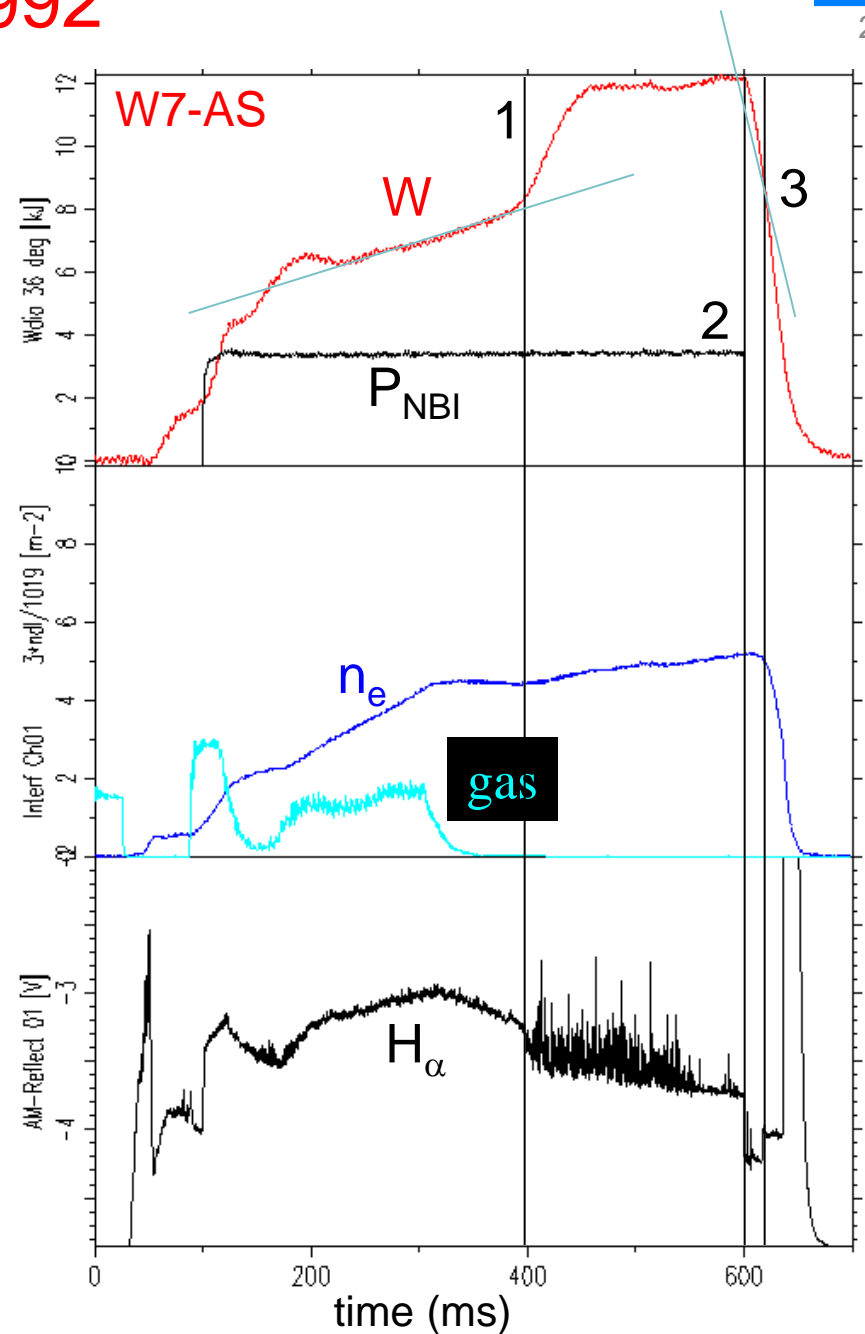


# The H-mode in stellarators: 1992



all main features reproduced

→ universality of the H-mode



# The physics of the H-mode

First ideas:

OH-confinement restored, but:

K. H. Burrell wrote in his 89 paper:

*“At plasma currents between 2.0 and 2.5 MA, we have found that energy confinement time in H-mode can exceed the saturated Ohmic confinement time by more than a factor of two...”*

K. H. Burrell et al., Plasma Phys. Control. Fusion 31 (1989) 1649.

C.M. Bishop analysed 1986 the role of the magnetic shear at the X-point on the stability of ballooning modes.

But JFT-2M produced a year later the H-mode in limiter configuration.

Many good ideas resting on “conventional” views on confinement and turbulence are summarized in the review

Plasma Phys. Control. Fusion **42** (2000) R1–R74. Printed in the UK

PII: S0741-3335(00)07020-2

REVIEW ARTICLE

2000

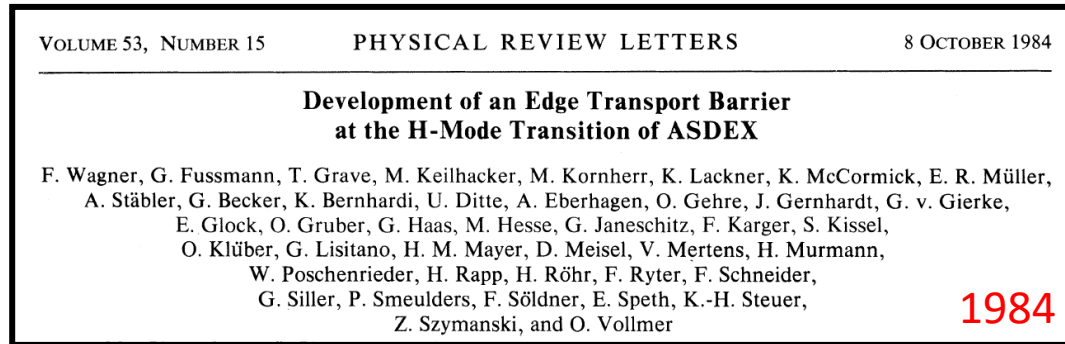
## **A review of theories of the L–H transition**

J W Connor and H R Wilson

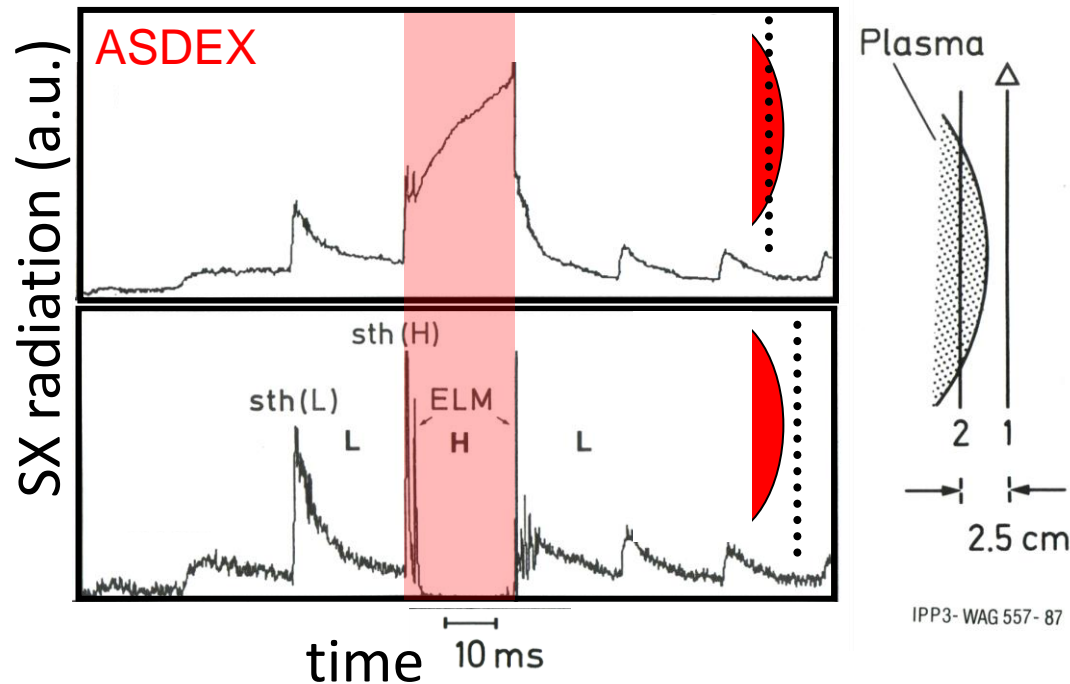
EURATOM/UKAEA Fusion Association, Culham Science Centre, Abingdon, Oxon, OX14 3DB,  
UK



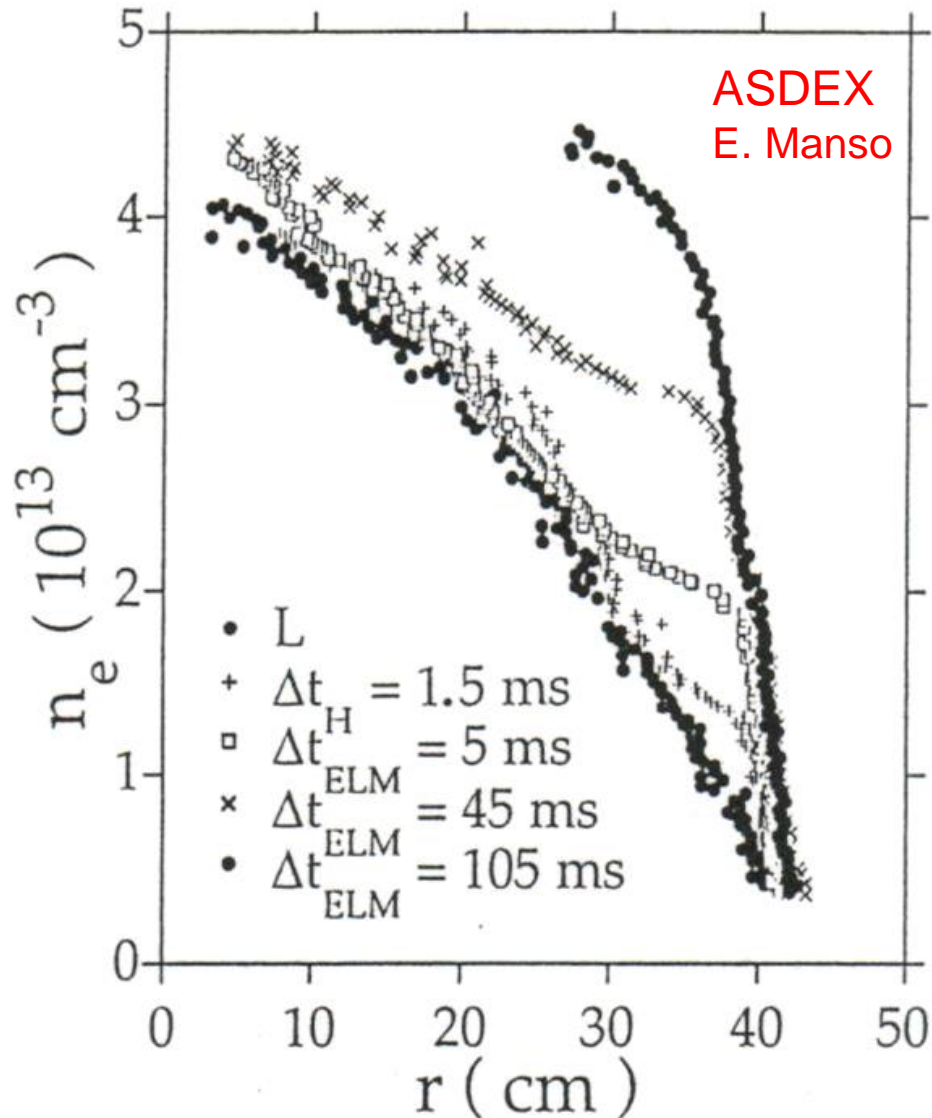
# 1. step in the understanding: edge transport barrier



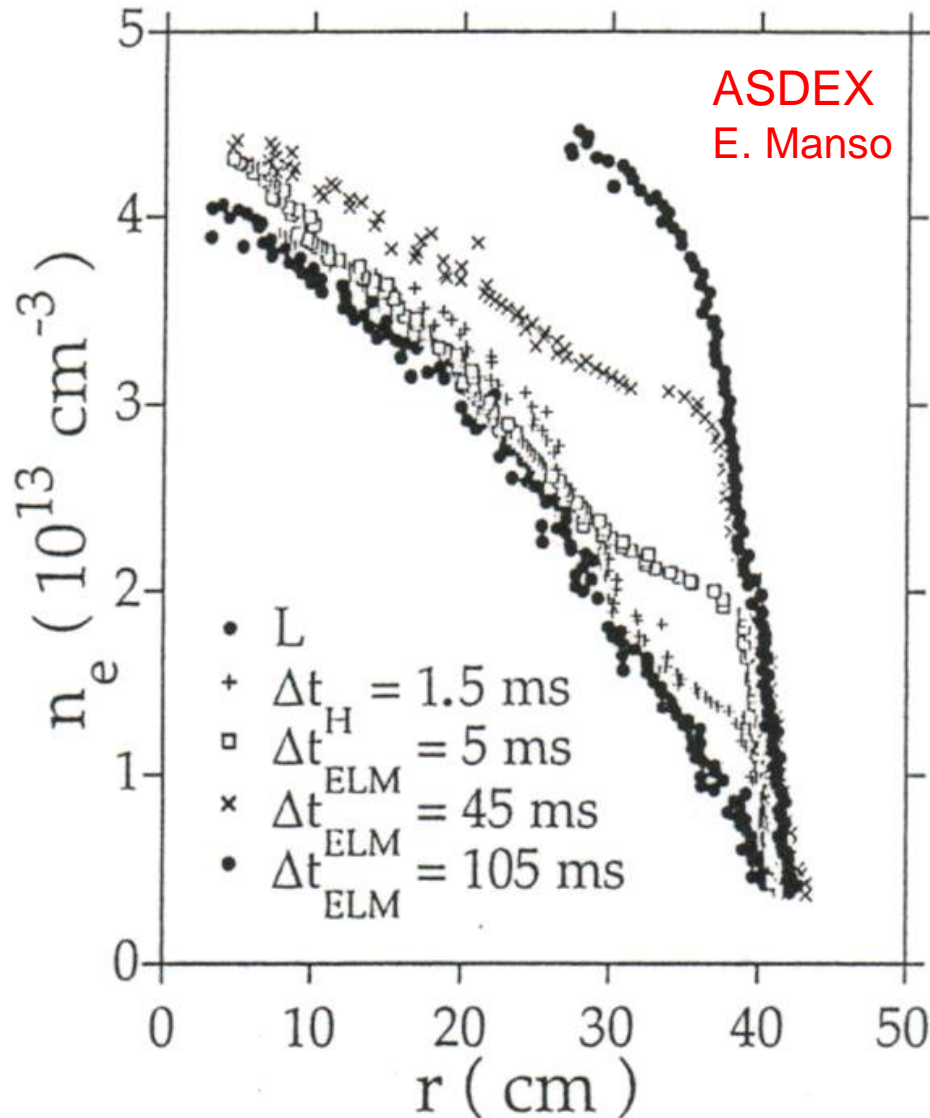
## Modulation of SX radiation at edge and in SOL via sawteeth



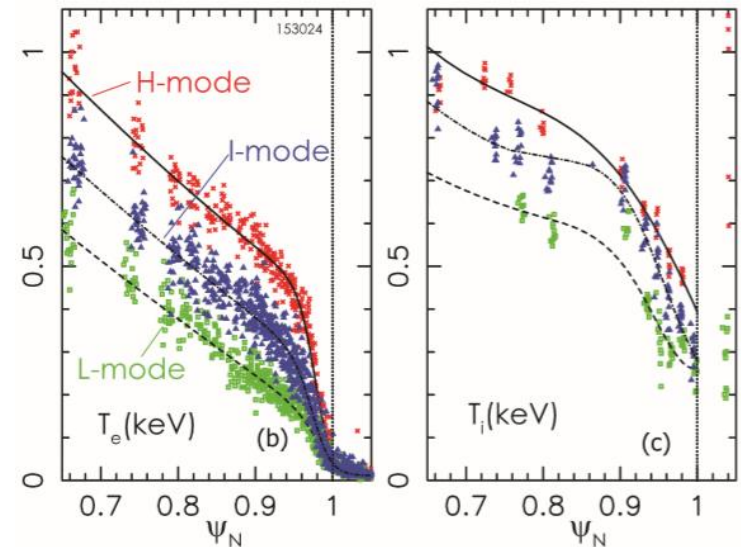
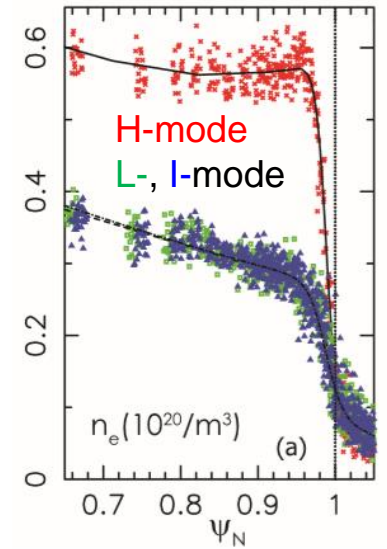
# H-mode feature: edge pedestals



# H-mode feature: edge pedestals



Edge pedestals  
of **DIII-D**





# When you google “edge pedestal and H-mode”

https://www.google.de/search?q=edge+pedestal+h-mode&tbm=isch&tbo=u&source=univ&sa=X&ved=0ahUKewiAm4KTqfHbAhVkJ0Jc 30%     Suchen

IPP email neu E-Mail @t-online.de FAZ.NET OZ - Ostsee-Zeitung Wikipedia LEO Verkehr - Service - ros... MVV Fahrplanauskunft Webcams Oberaudorf wetter.com FONIC Selfcare Deutschland im Überb...

The image displays a vast collection of search results for the query "edge pedestal and H-mode". The results are organized in a grid format, showing a wide variety of scientific content. Key elements include:

- Scientific Plots:** Numerous graphs showing profiles of plasma parameters (e.g., density, temperature, magnetic field) across the radial distance of a tokamak. Some plots show the characteristic "pedestal" structure at the plasma edge.
- Diagrams:** Schematic diagrams of tokamak cross-sections and edge regions, illustrating the location of the pedestal and H-mode characteristics.
- Images:** Photographs of laboratory equipment, such as diagnostic probes and camera setups, used for measuring the plasma edge.
- Logos and Text:** Logos for various research institutions and projects, including HEPP, DIII-D (University of California, San Diego), and others. Text snippets from abstracts or articles are also visible.
- Colorful Visuals:** Several plots use color gradients (e.g., red, blue, green) to represent different physical quantities, making the data more visually intuitive.

The overall impression is one of a rich and diverse scientific landscape, reflecting the complexity and interdisciplinary nature of plasma physics research in this area.

## 2<sup>nd</sup> step: $E_r$ enters magnetic confinement

$E_r$  is an implicit parameter in tokamak transport:  $\Gamma_e = \Gamma_i$  at continuous symmetry

First mention of  $E_r$  at the IAEA FE conference in 1984.

R. J. Taylor asks M. Keilhacker after his JET-talk:

*“Now, if it is a radial barrier, is it related to the radial electric field?”*

Stellarators are not continuously symmetric: An ambi-polar electric field enforces  $\Gamma_e = \Gamma_i$   
 $E_r$  plays an **explicit** role; development of roots (electron-, ion-root)

$E_r$  was introduced into H-mode theory by “stellarator people “ S.I. Itoh, K. Itoh and K.C. Shaing in 1989.



K.C.  
Shaing

## 2<sup>nd</sup> step: $E_r$ enters magnetic confinement

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K.C.  
Shaing

J. W. Connor and H. R. Wilson wrote in their 2000 NF paper:

### **A review of theories of the L-H transition**

*“Remarkably, changes in  $E_r$  at the transition were predicted theoretically (...) before they were observed experimentally; the observation of these has led to their inclusion in many later theories.”*



### 3. Step: The pioneering achievements of DIII-D

VOLUME 64, NUMBER 25

PHYSICAL REVIEW LETTERS

18 JUNE 1990

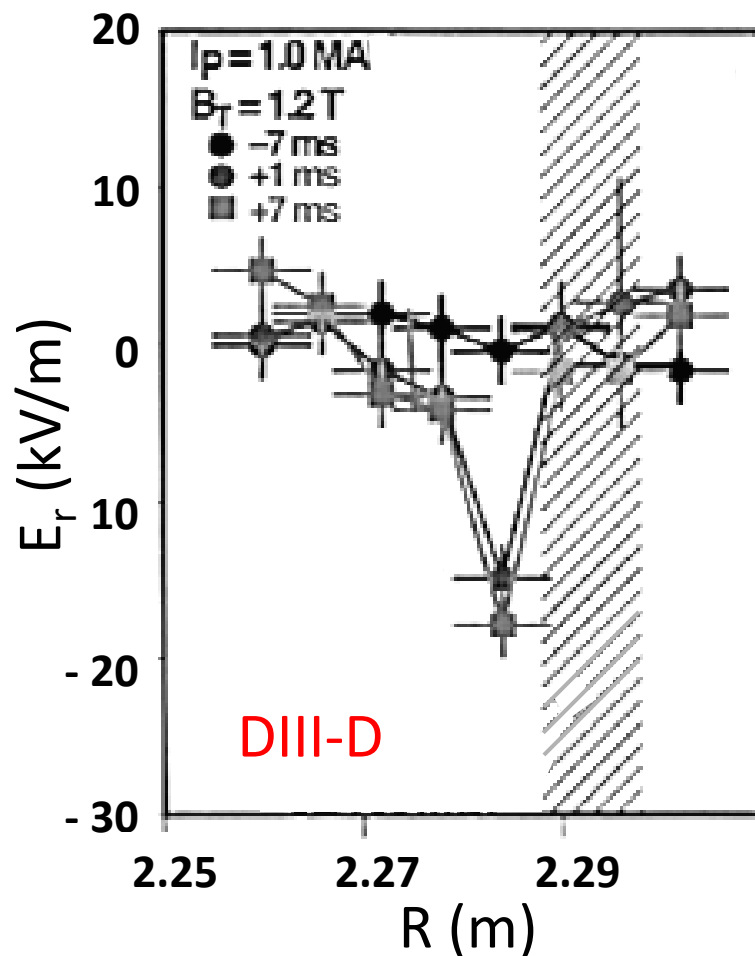
#### Role of Edge Electric Field and Poloidal Rotation in the *L-H* Transition

R. J. Groebner, K. H. Burrell, and R. P. Seraydarian

*General Atomics, San Diego, California 92138*

(Received 17 November 1989)

1990



First documentation of strong  $E_r$  change at the plasma edge

# The strong effect of the H-transition on $E_r$ (edge)

VOLUME 64, NUMBER 25

PHYSICAL REVIEW LETTERS

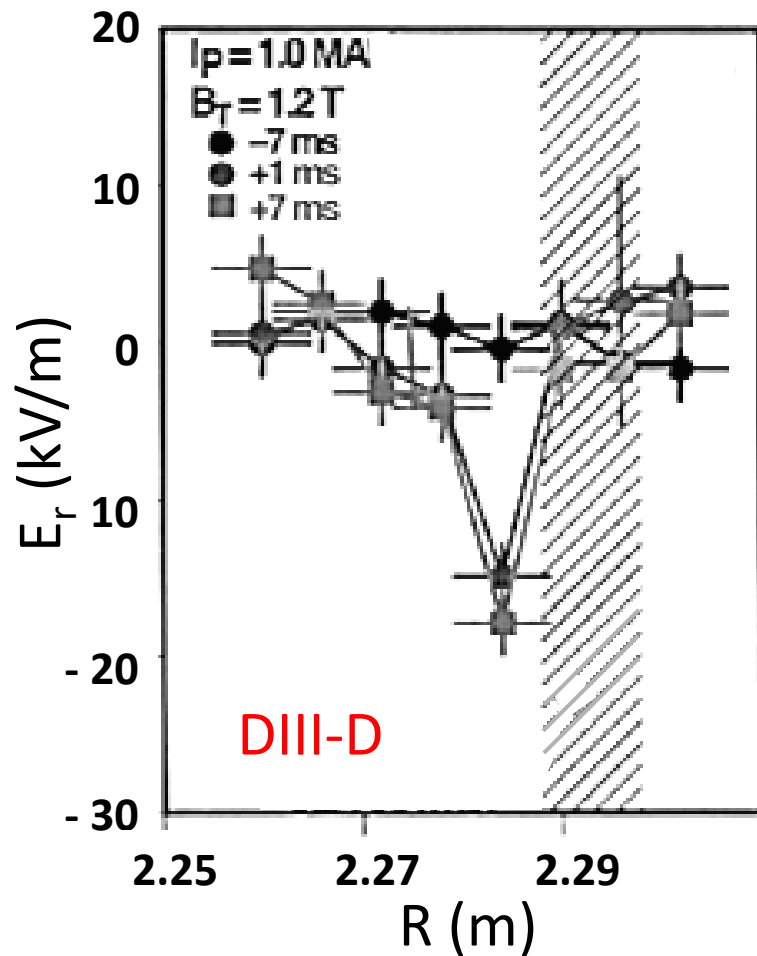
18 JUNE 1990

## Role of Edge Electric Field and Poloidal Rotation in the $L$ - $H$ Transition

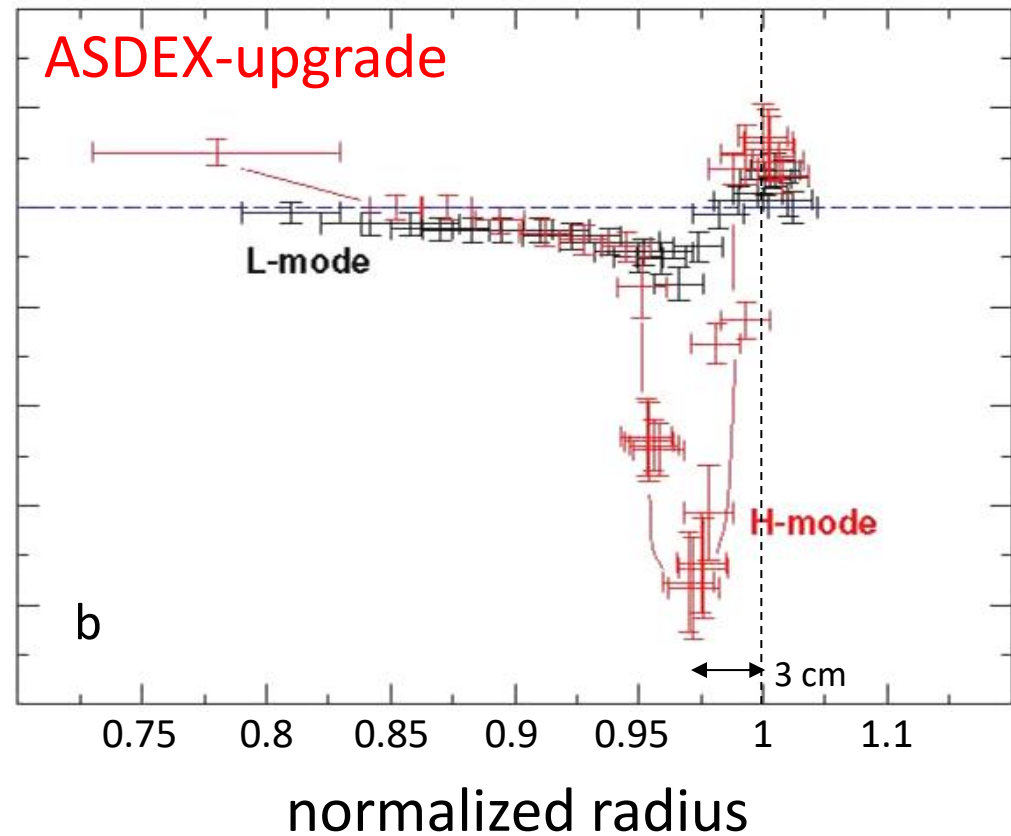
R. J. Groebner, K. H. Burrell, and R. P. Seraydarian

General Atomics, San Diego, California 92138

(Received 17 November 1989)



Nowadays with improved diagnostics



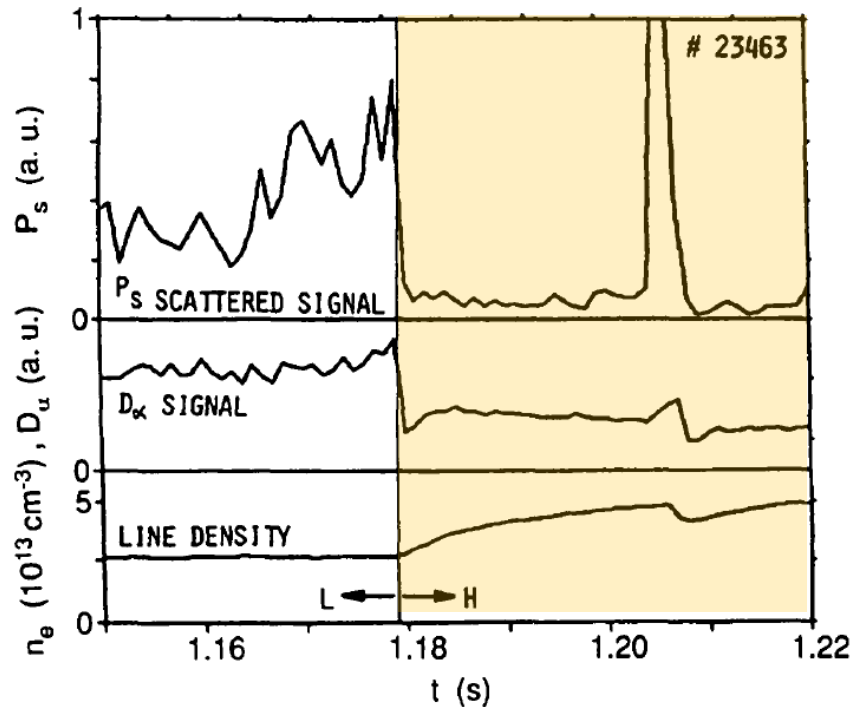


# Quiescent nature of the edge transport barrier

reduction of turbulence  
within transport barrier

reduction of the edge transport  
measured with Langmuir probes

ASDEX 1988



DIII-D 1995

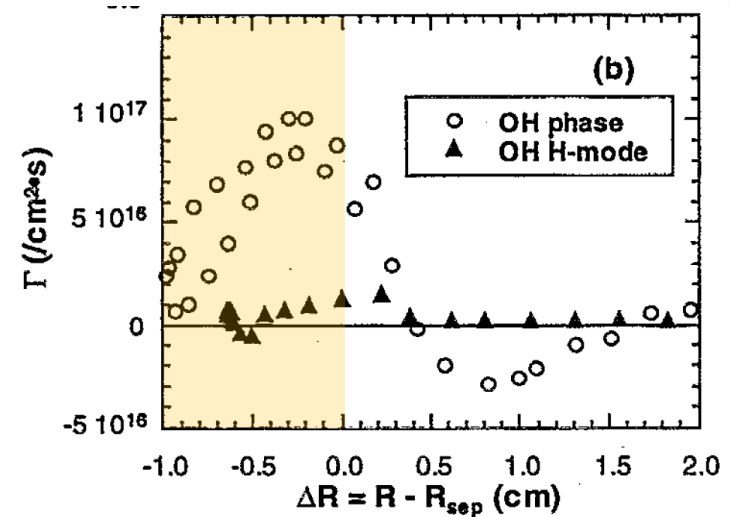


FIG. 80. Scattered signal power  $P_s$  measured close to the separatrix at the transition from an L-phase to a quiescent H-phase at  $k_{\perp} = 3 \text{ cm}^{-1}$  (outer vertical chord). Also plotted are the  $D_{\alpha}$  signal in the divertor and the electron line density 30 cm from the plasma centre.

Within the edge barrier:  
fluctuations and particle flux are strongly  
reduced

**whereas the gradients steepen**

# 4. Step - The BDT criterion, 1989

## Influence of sheared poloidal rotation on edge turbulence

H. Biglari and P. H. Diamond

*Department of Physics, University of California at San Diego, La Jolla, California 92093  
and General Atomics, San Diego, California 92138*

P. W. Terry

*Department of Physics, University of Wisconsin, Madison, Wisconsin 53706*

(Received 5 June 1989; accepted 20 October 1989)

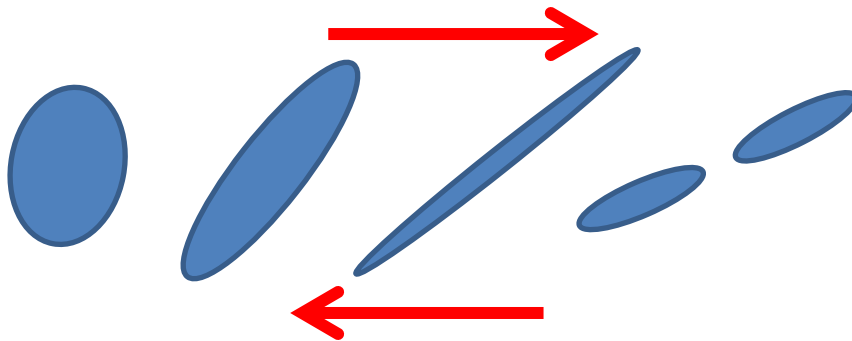


### Basic idea:

In a magnetised plasma  $E_r$  and  $E \times B$  flow are equivalent

The perpendicular flow is inhomogeneous, it has shear

The following process happens:



Turbulent eddies are tilted,  
stretched and strained out.

The turbulence structures are  
**decorrelated**  
leading to a lower turbulence level  
and reduced transport

# 4. Step - The BDT criterion, 1989

## Influence of sheared poloidal rotation on edge turbulence

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(Received 5 June 1989; accepted 20 October 1989)



Quantitatively:

The shearing rate  $>$  turbulence growth rate:  $\omega_{E \times B} > \gamma_{\text{lin}}^{\text{max}}$

$$\omega_{E \times B} \approx \frac{1}{B_{\text{tor}}} \frac{\partial E_r}{\partial r}$$

$$\gamma_{\text{lin}}^{\text{max}} \sim \frac{v_T}{L_T} \quad v_T = \sqrt{\frac{2T}{m}} \quad L_T = \frac{T}{\nabla T}$$

# 4. Step - The BDT criterion, 1989

## Influence of sheared poloidal rotation on edge turbulence

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(Received 5 June 1989; accepted 20 October 1989)



## Historical reminiscence, 1966:

THE PHYSICS OF FLUIDS

VOLUME 9, NUMBER 7

JULY 1966

### Short-Circuit of Flute Disturbances at a Plasma Boundary

B. LEHNERT

*Royal Institute of Technology, Stockholm, Sweden  
(Received 28 January 1966)*

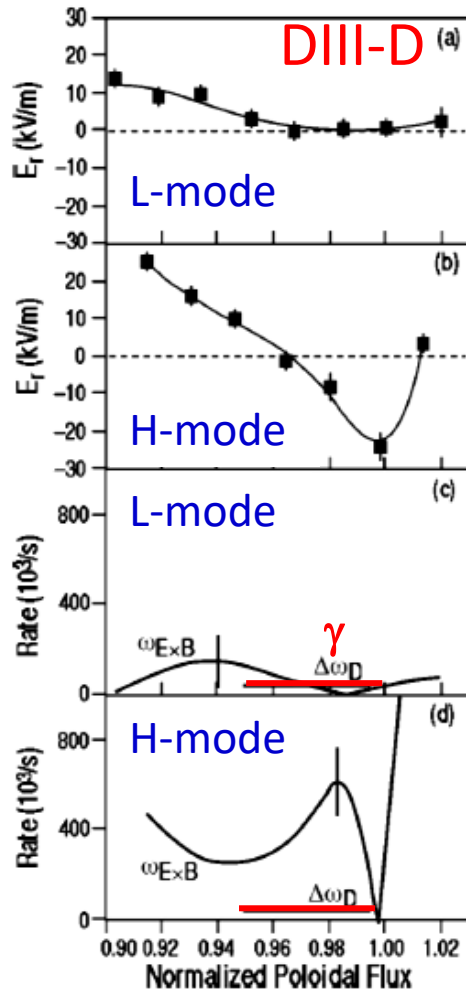
*“Thus, a non-uniform velocity should have a stabilizing tendency by “smearing out” the flute disturbance.”*

Bo Lehnert

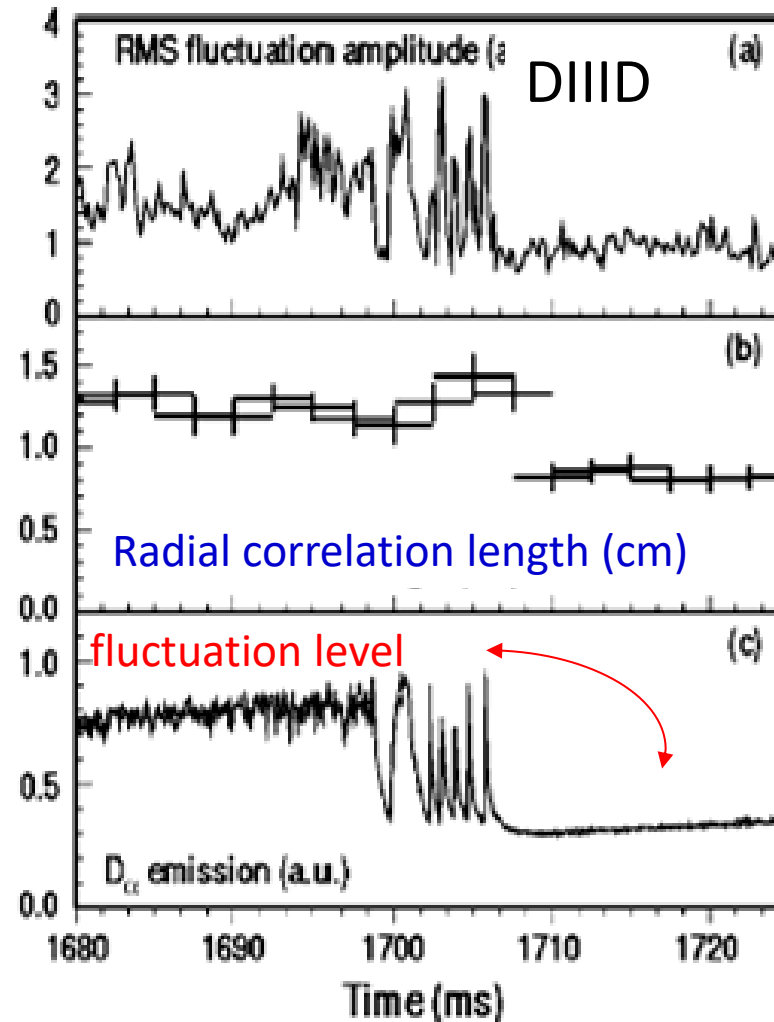


# 5. Step: Confirmation of the BDT criterion by DIII-D

Conditions for  
flow-decorrelation



Reduction of radial  
correlation length



# BUT: What is the origin of $E_r$ ?

The radial force balance enters:  $E_r = \frac{1}{Z_i e n_i} \frac{\partial p_i}{\partial r} + v_\Phi B_\theta - v_\theta B_\Phi$

energy  
balance  
with  
anomalous  
transport  
at the edge

momentum  
input  
and  
transport

poloidal  
force  
balance

Poloidal force balance:  $0 = j_r B/n - m_i \mu_\theta v_\theta + m_i \vartheta / \vartheta r (\langle \tilde{v}_r \tilde{v}_\theta \rangle)$

ion-losses  
momentum  
losses  
into SOL  
  
polarisation  
experiments

neo-classical  
damping  
  
specific  
for  
stellarators

turbulent  
Reynolds stress  
  
2D: spectral transport  
from small to large  
scales  
→ zonal flows

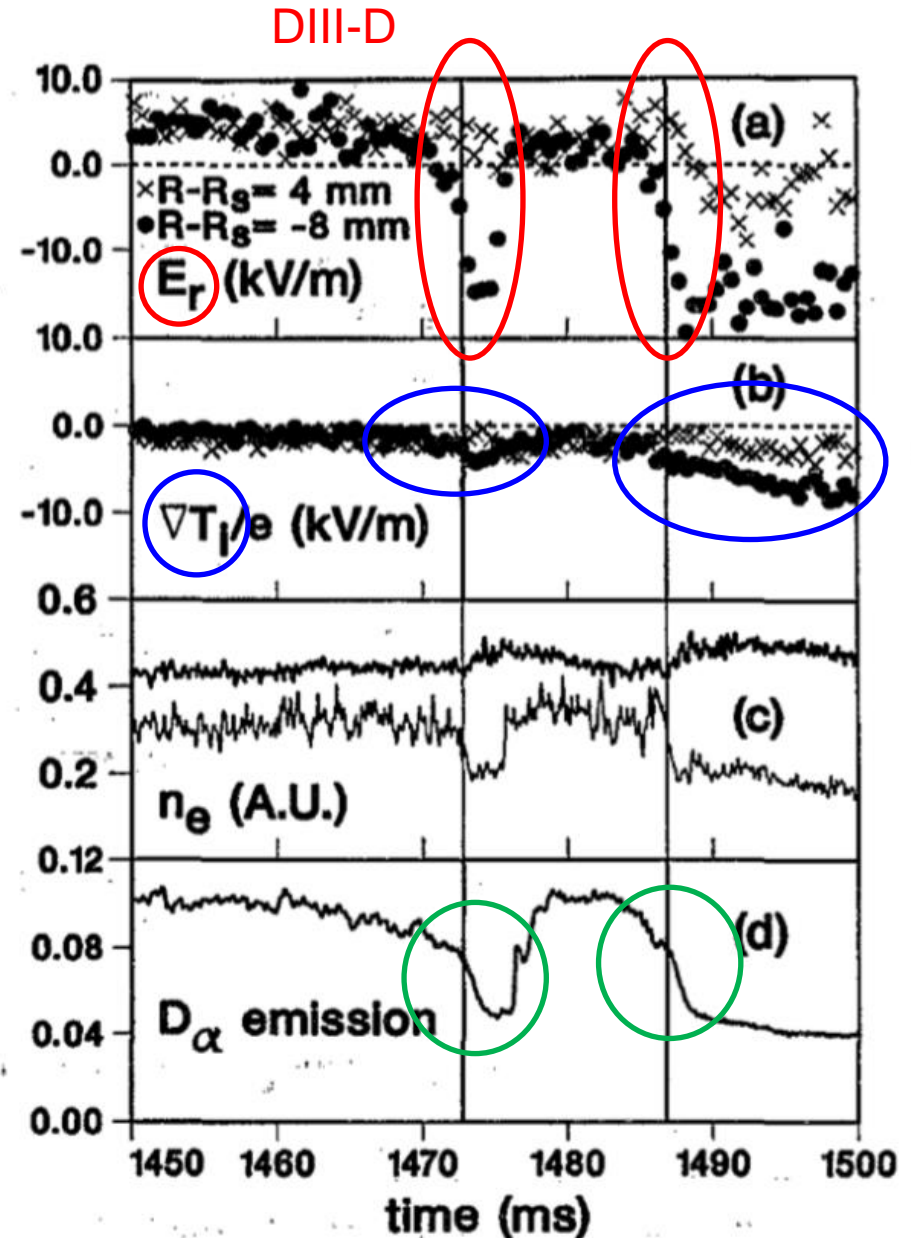
# Evidence for the $v \times B$ -term: 1994

**Beyond paradigm: Turbulence, transport, and the origin of the radial electric field in low to high confinement mode transitions in the DIII-D tokamak\***

R. A. Moyer,<sup>†</sup> K. H. Burrell,<sup>a)</sup> T. N. Carlstrom,<sup>a)</sup> S. Coda,<sup>b)</sup> R. W. Conn, E. J. Doyle,<sup>c)</sup>  
 P. Gohil,<sup>a)</sup> R. J. Groebner,<sup>a)</sup> J. Kim,<sup>a)</sup> R. Lehmer, W. A. Peebles,<sup>c)</sup> M. Porkolab,<sup>b)</sup>  
 C. L. Rettig,<sup>c)</sup> T. L. Rhodes,<sup>c)</sup> R. P. Seraydarian,<sup>a)</sup> R. Stockdale,<sup>a)</sup> D. M. Thomas,<sup>a)</sup>  
 G. R. Tynan,<sup>c)</sup> and J. G. Watkins<sup>c)</sup>  
 Fusion Energy Research Program, University of California, San Diego, La Jolla, California 92093-0417

(Received 14 November 1994; accepted 6 March 1995)

$E_r$  changes faster  
 than  
 $\nabla p_i$  at the edge

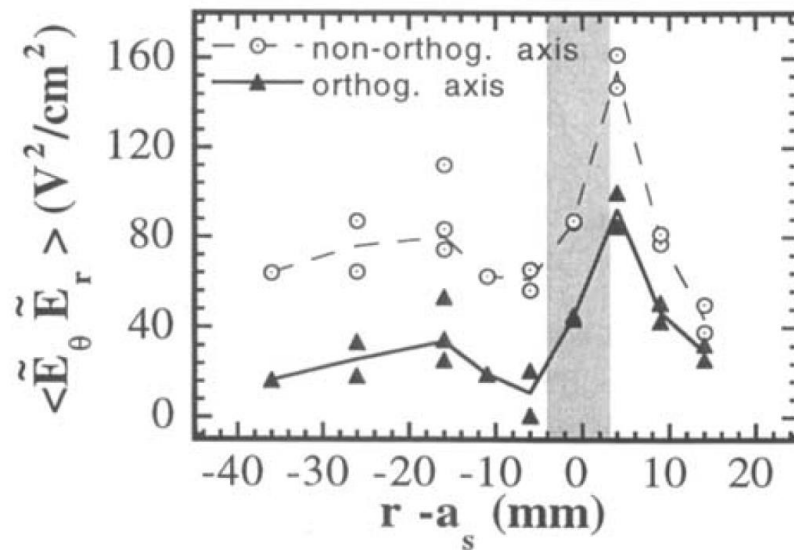




# Evidence for Reynolds stress

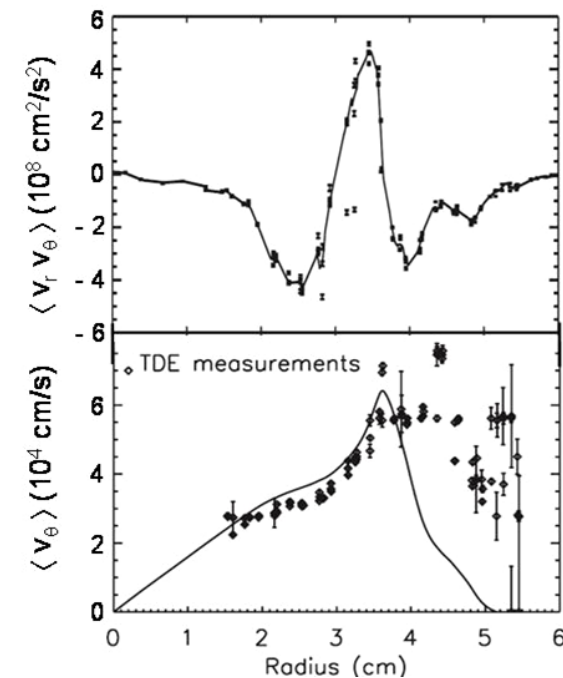
Reynolds stress  
in the shear layer of TJ-II  
C. Hidalgo et al.

2000



Reynolds stress  
leads to st.st. flow  
and  $E_r$   
C. Holland et al.

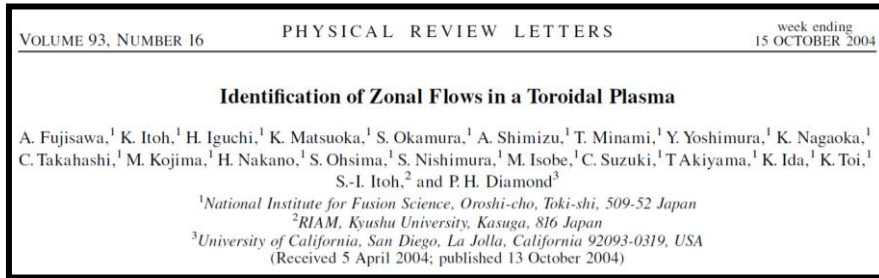
CSDX  
2006



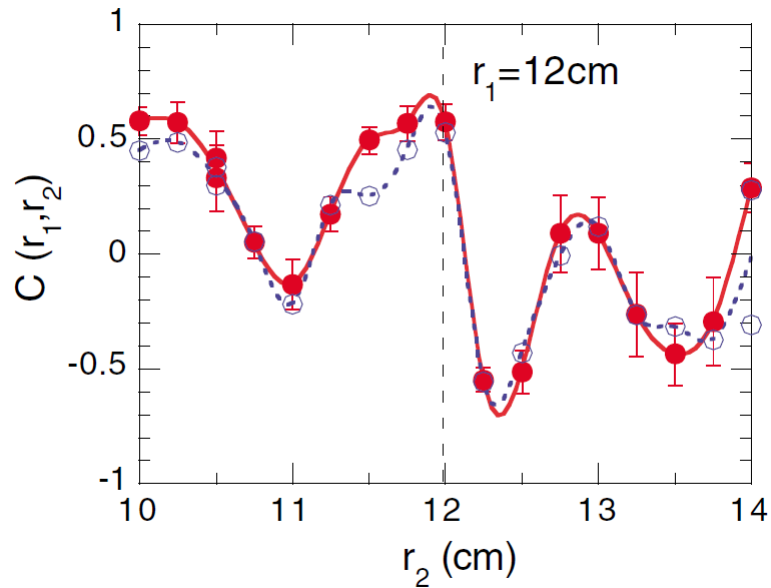
Status now: turbulence causes Reynolds stress causing  $E \times B$  flow in the form of zonal flows which acts back on and regulates the level of turbulence



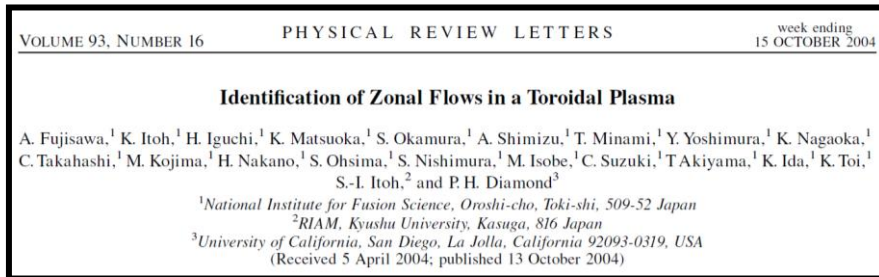
# Induced flows in two forms: ZFs and GAMs



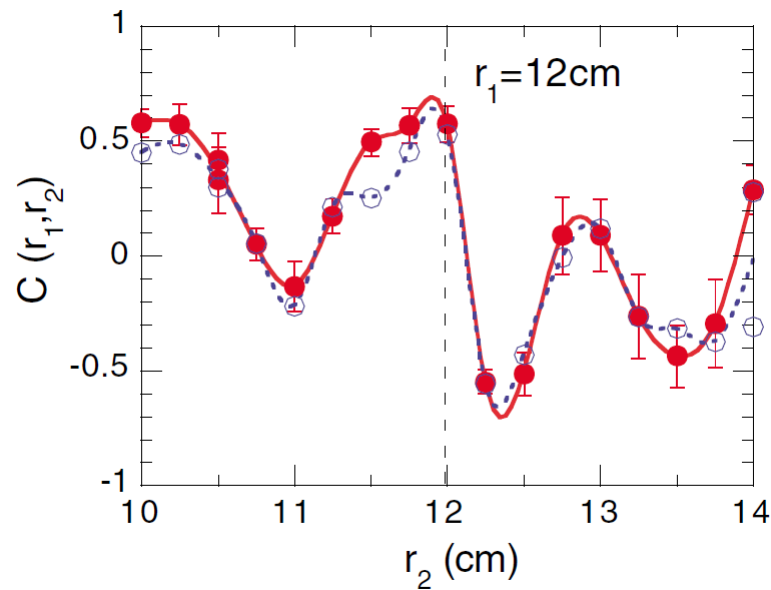
## Radial structure of a zonal flow, CHS 2004



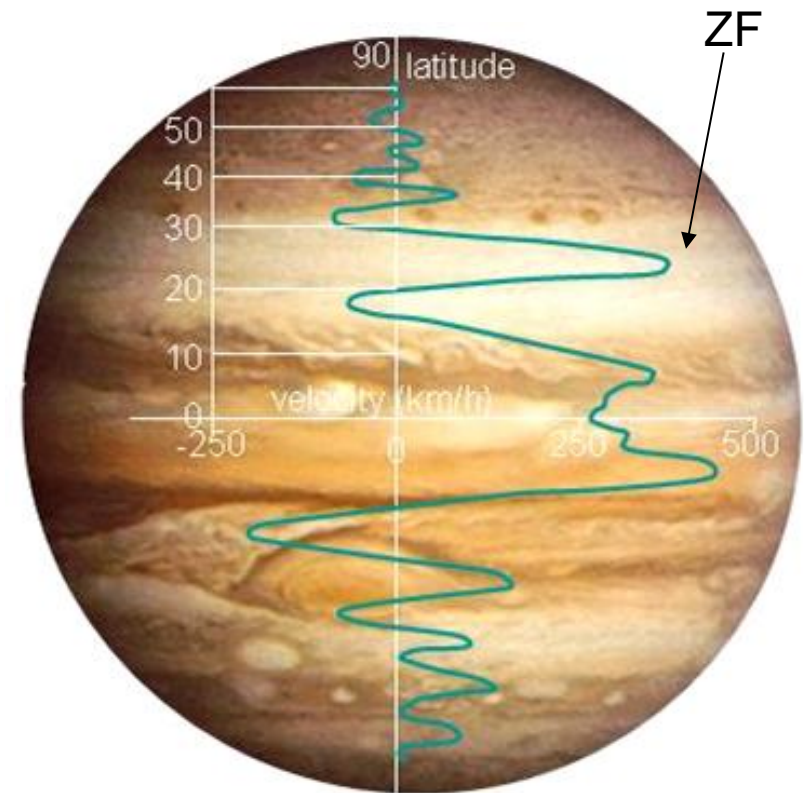
# Induced flows in two forms: ZFs and GAMs



Radial structure of a zonal flow, CHS 2004



The beauty of this physics  
Planet Jupiter



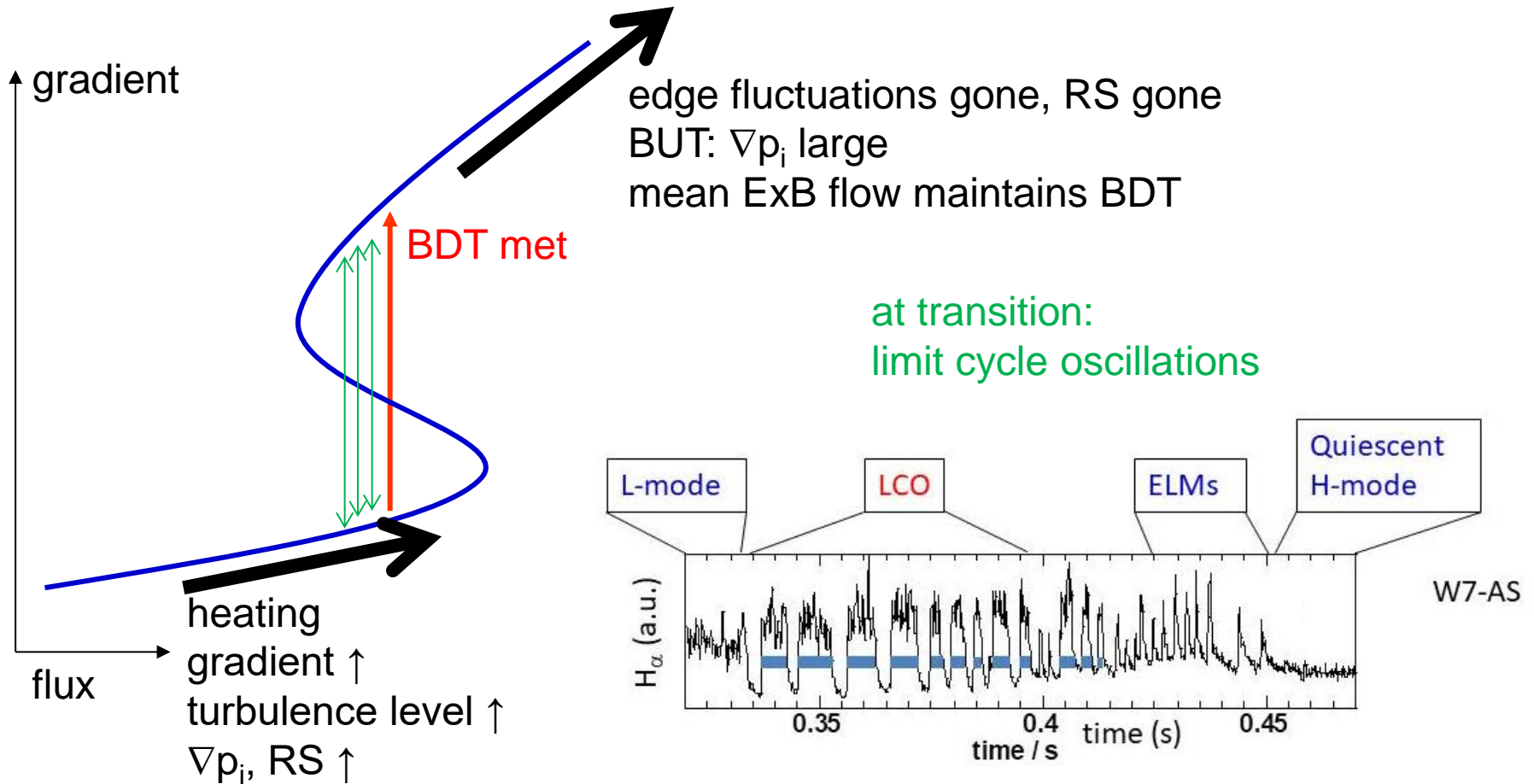
# Visualisation of spectral transport in 2D turbulence

2D model system using an electrolyte and  $\mathbf{j} \times \mathbf{B}$  forces to drive eddies



A. Shats, ANU, Canberra

# General concept: The L-H transition as a bifurcation



Formal treatment of L-H transition: predator-prey complex

PRL 108, 155002 (2012) PHYSICAL REVIEW LETTERS week ending 13 APRIL 2012

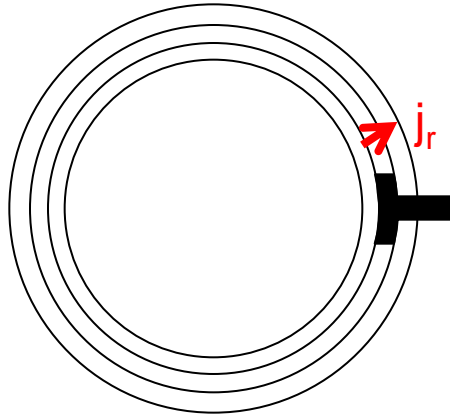
**Role of Zonal Flow Predator-Prey Oscillations in Triggering the Transition to H-Mode Confinement**

L. Schmitz,<sup>1</sup> L. Zeng,<sup>1</sup> T.L. Rhodes,<sup>1</sup> J.C. Hillesheim,<sup>1</sup> E.J. Doyle,<sup>1</sup> R.J. Groebner,<sup>2</sup> W.A. Peebles,<sup>1</sup> K.H. Burrell,<sup>2</sup> and G. Wang<sup>1</sup>

2012

# The causality chain: induced H-mode of TEXTOR

plasma  
biasing

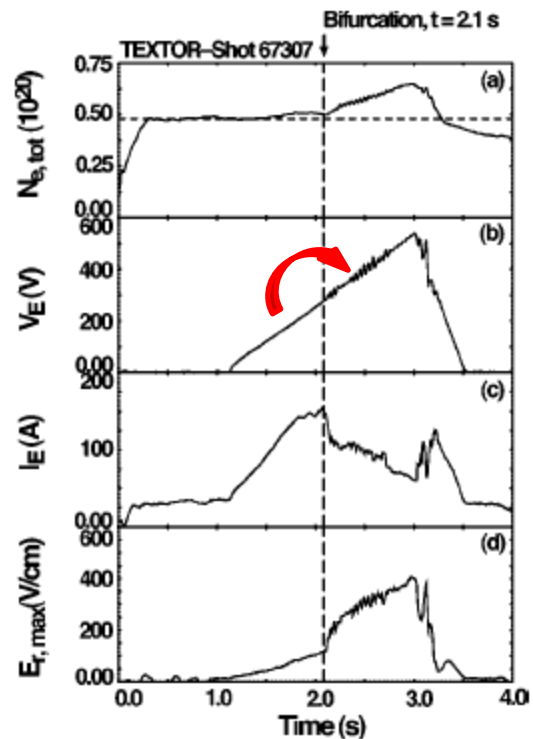


particle  
content

biasing  
voltage

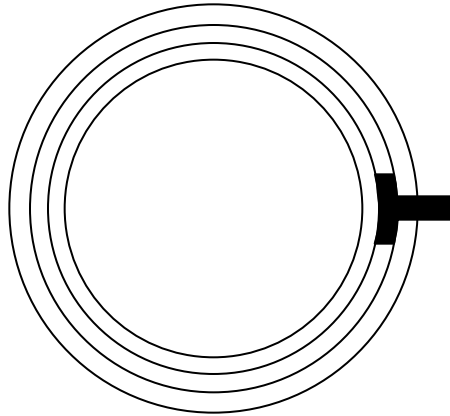
electrode  
current

max  $E_r$



# The causality chain: induced H-mode of TEXTOR

plasma  
biasing



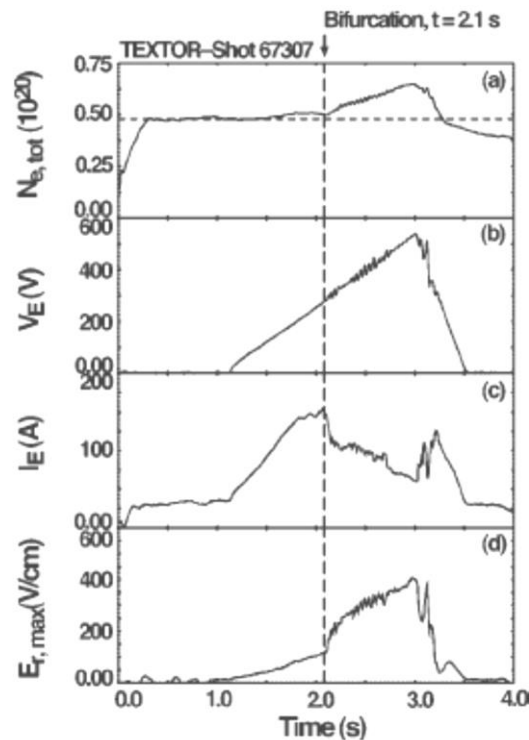
particle  
content

biasing  
voltage

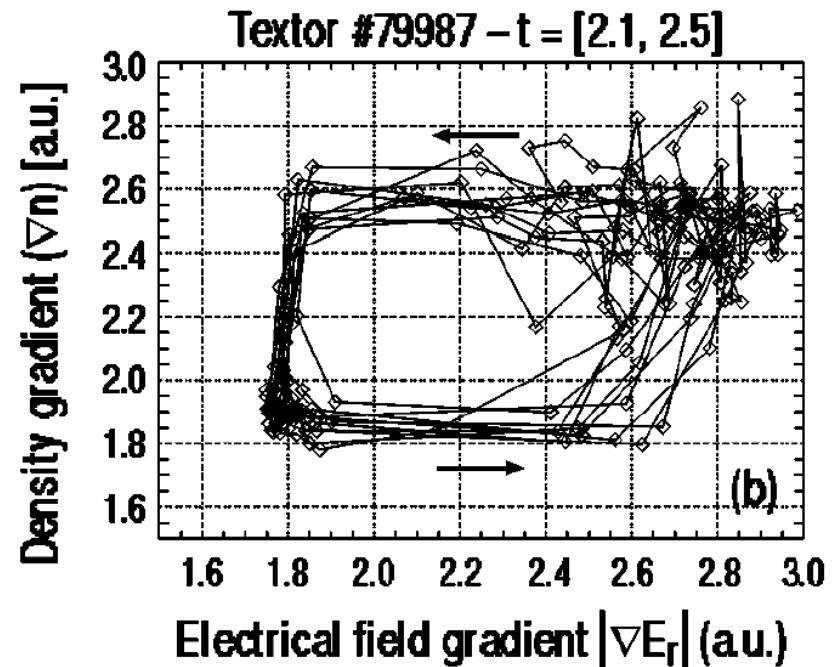
electrode  
current

max  $E_r$

1990



Lissajous orbit



voltage is sinusoidally modulated  
 $E_r$ -field gradient change leads  
 density gradient follows after  $\sim 5$  ms

# Possible course of action from L- to H-mode

Increase of heating power toward power threshold

Edge gradients and fluctuation level increases in the L-phase

Asymmetry in poloidal fluctuation pattern allows the turbulent Reynolds stress to develop  
e.g. via separatrix X-point, magnetic shear

The neo-classical mean-flow  $E_r$ -field is supported by the  $\partial/\partial r(\langle v_r v_\theta \rangle)$ -term

In the marginal state, the edge jumps between L- and H-mode (LCO)

The  $E_r$  field reduces the fluctuation level and the Reynolds term disappears again

The neoclassical  $E_r$ -field is large enough to stabilize the H-mode

**Question:** is a short-cut possible, without Reynolds Stress as mediator?

Obviously in case of ASDEX-Upgrade (M. Cavedon *et al* 2017 *Nucl. Fusion* 57 014002)

## General observations:

The response of a fusion plasma as a thermodynamically open system to increased power input can be a reduced level of turbulence.

Anomalous transport is a highly non-linear problem with self-regulating mechanisms, which can lead to the lowest dissipation state – collisional transport



# Improving the H-mode

## Several strategies:

improved edge stability (increase edge pedestal)

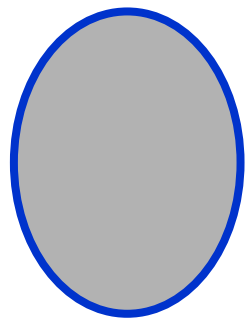
avoiding, suppressing ELMs

via RMP (resonant magnetic perturbations)

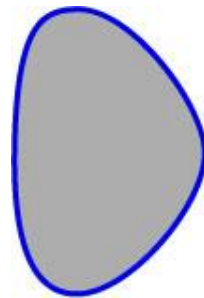
selected operational windows: ELMs are replaced by quasi-coherent edge instabilities

expanding edge pedestal

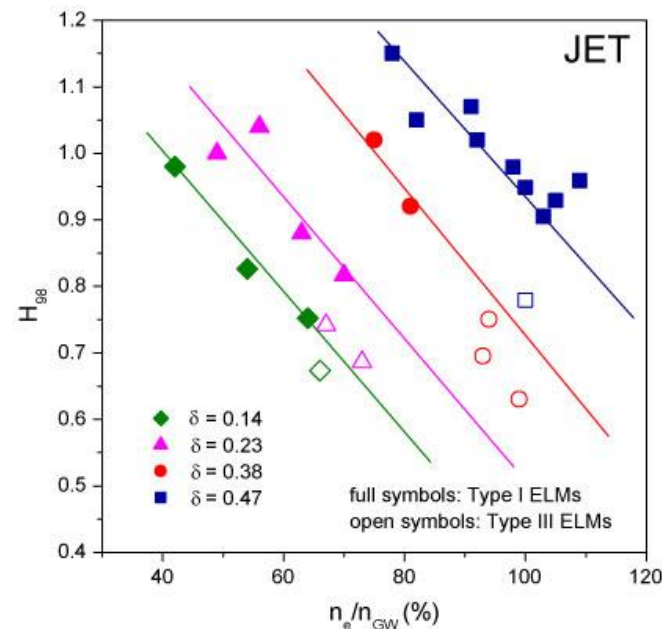
internal transport barriers (ITBs)



elongated



triangular  $\delta$



2003



# Improving the H-mode

## Expanding the barrier

VH-mode of DIII-D, 1991

VOLUME 67, NUMBER 22

PHYSICAL REVIEW LETTERS

25 NOVEMBER 1991

### Regime of Very High Confinement in the Boronized DIII-D Tokamak

G. L. Jackson, J. Winter,<sup>(a)</sup> T. S. Taylor, K. H. Burrell, J. C. DeBoo, C. M. Greenfield, R. J. Groebner, T. Hodapp, K. Holtrop, E. A. Lazarus,<sup>(b)</sup> L. L. Lao, S. I. Lippmann, T. H. Osborne, T. W. Petrie, J. Phillips, R. James,<sup>(c)</sup> D. P. Schissel, E. J. Strait, A. D. Turnbull, W. P. West, and DIII-D Team

*General Atomics, San Diego, California 92128*

(Received 23 August 1991)

high density ( $n_e(0) \sim 10^{20} \text{ m}^{-3}$ )  
high  $T_i$  (13.6 keV)

H-factor: 3.5 !

## ELMs replaced by quasi-coherent edge instability

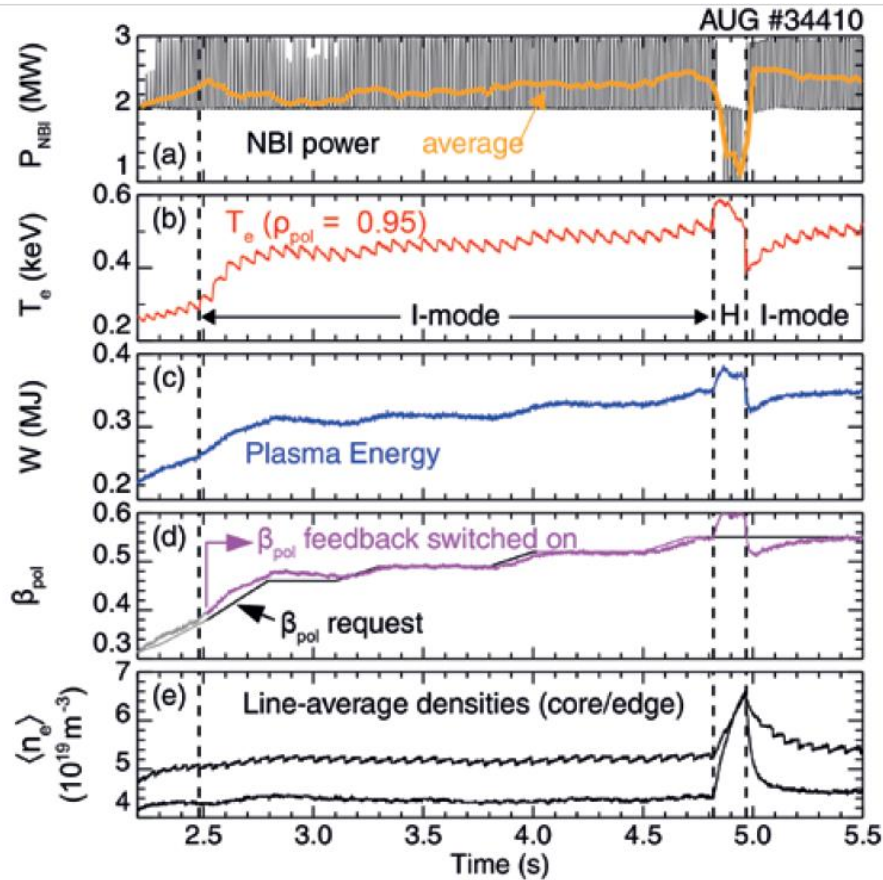
EDA-mode of Alcator C-mod  
with QC-edge mode 2000

QH-mode of DIII-D  
with EHO at the edge 2002

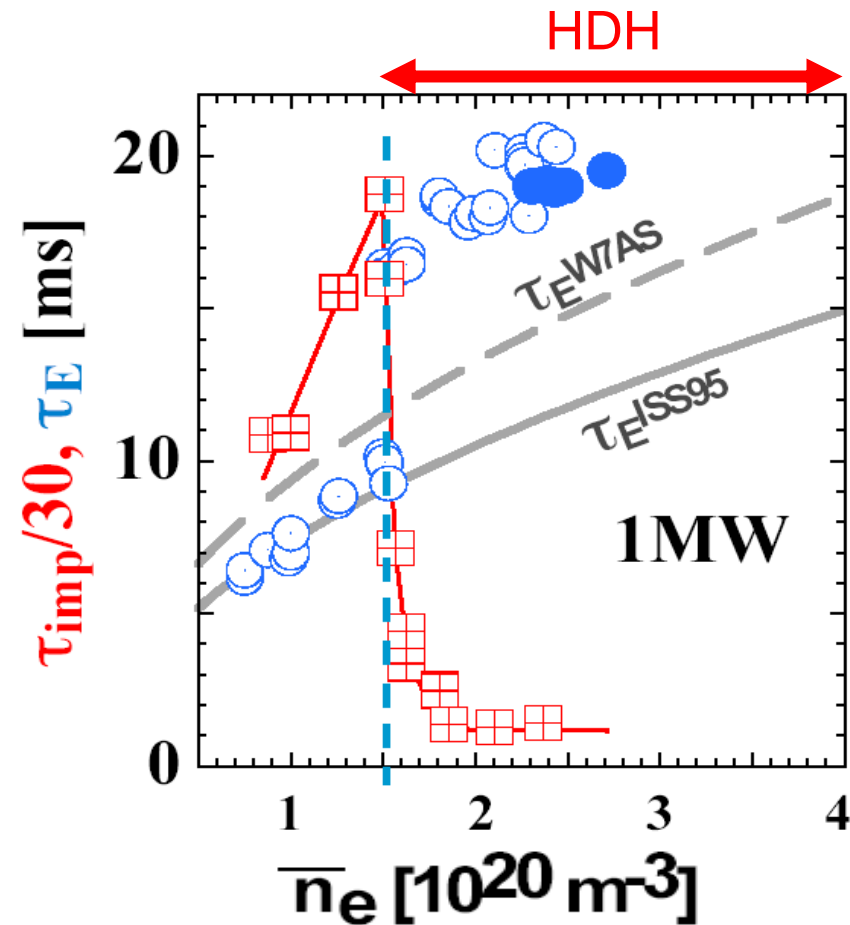
I-mode of Alcator C-mod  
with WCM at the edge 2010

# Improving the H-mode

## Stationary I-mode of A-UP



## HDH regime of W7-AS, 2002



# Improving the H-mode

## Internal transport barriers

of JT-60 U, 1994

VOLUME 72, NUMBER 23

PHYSICAL REVIEW LETTERS

6 JUNE 1994

### Internal Transport Barrier on $q = 3$ Surface and Poloidal Plasma Spin Up in JT-60U High- $\beta_p$ Discharges

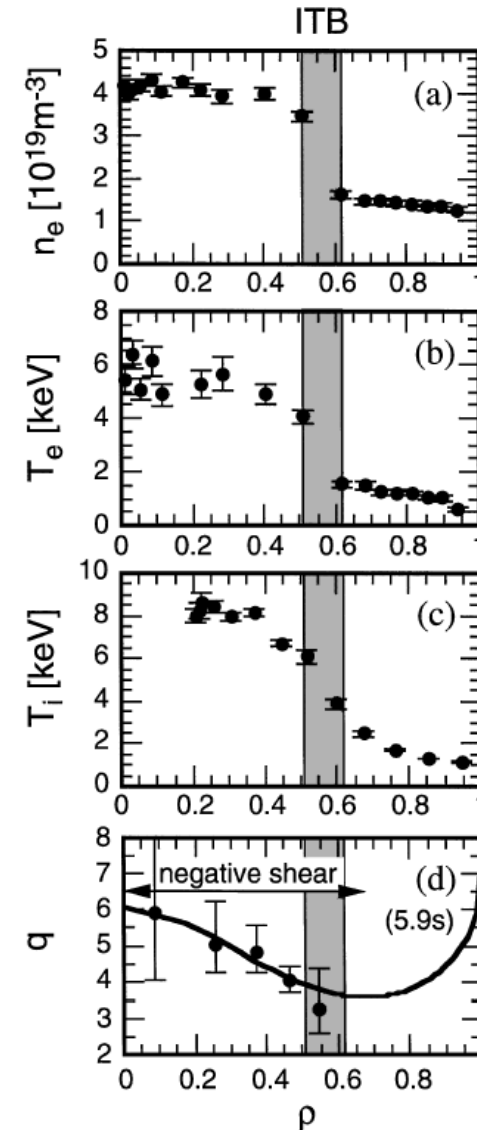
Y. Koide, M. Kikuchi, M. Mori, S. Tsuji, S. Ishida, N. Asakura, Y. Kamada, T. Nishitani, Y. Kawano,  
T. Hatae, T. Fujita, T. Fukuda, A. Sakasai, T. Kondoh, R. Yoshino, and Y. Neyatani  
*Japan Atomic Energy Research Institute, Naka Fusion Research Establishment,  
Naka-machi, Naka-gun, Ibaraki-ken 311-01, Japan*  
(Received 27 May 1993)

Tayloring of core- $q$ -profile necessary  
by flattening it or even reverse it

lowish  $I_p$ , high  $\beta_{pol}$   
off-axis heating  
high bootstrap current fraction

Achievement in JT-60 U:

**highest  $Q_{DD}$**



T. Fujita, PRL 78, 2377, 1997

# Challenges

predict  $\tau_E$  of ITER  
good edge confinement without ELMs....

What I am interested in - 10 years after retirement:

Modelling of an H-transition without outside trigger

Understanding of the isotope effect

effect is known since 1973 (TFR)

first scaling: J. Hugill and J. Sheffield 1978 Nucl. Fusion 18 15

(devices for scaling studies: T3, T4, ST, Cleo, Ormak, Pulsator, TFR, Alcator, Tosca, PLT, T10)

is there an isotopic effect in stellarators?

Replacement of gas fuelling by pellets (or at least supersonic injection)

Physics of the current scaling.

tokamak scaling:

$$\tau_E \text{ (L-mode, ITER 89)} \sim I_p^{0.85} B^{0.2} R^{1.2} a^{0.3} \kappa^{0.5} A^{0.5} n_e^{0.1} P^{-0.5}$$

$$\tau_E \text{ (H-mode, ITER 98 y2)} \sim I_p^{0.9} B^{0.2} R^{1.4} a^{0.6} \kappa^{0.8} A^{0.2} n_e^{0.4} P^{-0.7}$$

stellarator scaling:

$$\tau_E \sim I^{0.4} B^{0.8} R^{0.7} a^2 n_e^{0.5} P^{-0.6}$$

$$\text{with } I \sim I_p R/a^2 B \rightarrow I_p^{0.4} B^{0.4} R^{1.3} a^{1.2}$$

# Thank you very much

Thanks to those who organised the connection